UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

MAR 11 1981

In Reply Refer To:
RII:JPO
50-491, 50-492
50-493, 50-488
50-489, 50-490

Duke Power Company
ATTN: L. C. Dail, Vice President
Design Engineering
P. O. Box 33189
Charlotte, NC 28242

Gentlemen:

The enclosed IE Information Notice No. 81-06 contains information that may be applicable to your facility regarding the failure of an ITE Model K-600 circuit breaker. No specific action or response is requested at this time; however, pending the results of NRC staff evaluations of this matter, further licensee actions may be requested.

Should you have any questions regarding this information notice, please contact this office.

Sincerely,

Robert D. Martin for

James P. O'Reilly
Director

Enclosures:
1. IE Information Notice No. 81-06
2. List of Recently Issued
IE Information Notices

cc w/encl:
J. T. Moore, Project Manager

SSINS No.: 6835
Accession No.:
8011040269
IN 81-06

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

March 11, 1981

IE Information Notice No. 81-06: FAILURE OF ITE MODEL K-600 CIRCUIT BREAKER

Purpose:

The purpose of this information notice is to provide early notification of a matter that could adversely affect the safety of nuclear power plants. Toward this end, this information notice alerts holders of operating licenses and construction permits of a possibly generic deficiency involving Model K-600 circuit breakers manufactured by ITE.

Description of Circumstance:

By letter dated February 20, 1981, the Sacramento Municipal Utility District informed us that, while performing preventive maintenance on electrical circuit breakers at its Rancho Seco facility, an ITE Model K-600 breaker was observed not to trip. An investigation revealed that a tripping coil wire had slipped out of its terminal. Further examinations revealed that the coil wire size was AWG 20, whereas the mating lug was sized for AWG 16-14 wire. A check of other ITE electrical breakers indicated that the problem may be generic. Subsequent to the letter, the Sacramento Municipal Utility District informed us orally that ITE Model K-1600 breakers have the same deficiency and that Model K-3000 breakers may be similarly affected.

The information provided by the Sacramento Municipal Utility District is preliminary and will be followed by a complete report. Consequently, we have not yet evaluated the safety significance of this event. Nevertheless, the information herein is being provided as an early notification of a possibly significant event. Although no specific action or response is requested at this time, recipients should review the information for possible applicability to their facilities. If NRC evaluations so indicate, further licensee actions may be requested.

If you have any questions regarding the information contained herein, please contact the Director of the appropriate NRC Regional Office.

Attachment:
Recently issued IE Information Notices

Attachment
IN 81-06
March 11, 1981

RECENTLY ISSUED
IE INFORMATION NOTICES

Information Notice No.	Subject	Date of Issue	Issued to
81-05	Degraded DC System at Palisades	2/10/81	All power reactor facilities with an OL or CP
81-04	Cracking in Main Steam Lines	2/27/81	All power reactor facilities with an OL
81-03	Checklist for Licensees Making Notifications of Significant Events in Accordance with 10 CFR 50.72	2/12/81	All power reactor facilities with an OL
81-02	Transportation of Radiography Devices	1/23/81	All Radiography licensees
81-01	Possible Failures of General Electric Type HFA Relays	1/16/81	All power reactor facilities with an OL or CP.
80-45	Potential Failure of BWR Backup Manual Scram Capability	12/17/80	All PWR facilities with an OL or CP
80-44	Actuation of ECCS in the Recirculation Mode While in Hot Shutdown	12/16/80	All PWR facilities with an OL or CP
80-43	Failures of the Continuous Water Level Monitor for the Scram Discharge Volume at Dresden Unit No. 2	12/5/80	All power reactor facilities with an OL or CP
80-42	Effect of Radiation on Hydraulic Snubber Fluid	11/24/80	All power reactor facilities with an OL or CP
80-41	Failure of Swing Check Valve in the Decay Heat Removal System at Davis-Besse Unit No. 1	11/10/80	All power reactor facilities with an OL or CP

OL = Operating Licenses
CP = Construction Permits