UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VERMONT YANKEE NUCLEAR POWER CORPORATION

DOCKET NO. 50-271

VERMONT YANKEE NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63
License No. DPR-28

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application by Vermont Yankee Nuclear Power Corporation (the licensee) dated September 12, 1980, as supplemented January 5, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility License No. DPR-28 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 2, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. DPR-28

DOCKET NO. 50-271

Revise Appendix A as follows:

Remove	Insert
49	49
-	49a
60	60
61	61
190a	190a
193	193
215	215

TABLE 3.2.6

POST-ACCIDENT INSTRUMENTATION

Minimum Number of Operable Instrument Channels	Parameter	Type of Indication	Instrument Range
2	Drywell Atmospheric Temperature (Note 1)	Recorder #16-19-45 Recorder #TR-1-149	0-300° F 0-300° F
2	Drywell Pressure (Note 1) Torus Pressure (Note 1)	Recorder #16-19-44	0-80 psia 0-80 psia
2	Torus Water Level (Note 3)	Meter #16-19-46A Meter #16-19-46B	0-3 ft. 0-3 ft.
2	Torus Water Temperature	Meter #16-19-48	60-180° F
2	Reactor Pressure (Note 1)	Recorder #6-97 Meter #6-90A Meter #6-90B	0-1200 psig 0-1200 psig 0-1200 psig
2	Reactor Vessel Water Level (Note 1)	Meter #2-3-91A Meter #2-3-91B	(-150)-0-(+150)" H_2O (-150)-0-(+150)" H_2O
1	Control Rod Position (Note 1, 2)	Meter	0-48" RPIS
1	Neutron Monitor (Note 1, 2)	Meter	0-125% Rated Flux
1	Torus Air Temperature (Note 1)	Recorder #TR-16-19-45	0-300° F
2/valve	Safety/Relief Valve Position from pressure switches (Note 4)	Lights (SRV 2-71-A through D)	Closed - Open
1/valve	Safety Valve Position from Acoustic Monitor (Note 5)	Meter Z1-2-1A/B	Closed - Open

Note 1 - From and after the date that one of these parameters is not indicated in the control room, continued reactor operation is permissible during the next seven days. If reduced to one indication of a parameter operation is permissible for 30 days.

Note 2 - Control rod position and neutron monitor instruments are considered to be redundant to each other.

TABLE 3.2.6 NOTES

Note 3 - From and after the date that this parameter is reduced to one indication in the control room, continued reactor operation is permissible during the next thirty days. If both channels are inoperable and indication cannot be restored in six hours, an orderly shutdown shall be initiated and the reactor shall be in a hot shutdown condition in six hours and a cold shutdown condition in the following 18 hours.

Note 4 - From and after the date that safety/relief valve position from pressure switches is unavailable, reactor operation may continue provided safety/relief valve position can be determined from recorder 2-166 (thermocouple, 0-600°F) and meter 16-19-48 (torus water temperature, 60-180°F). If both indications are not available, the reactor shall be in a hot shutdown condition in six hours and a cold shutdown condition in the following 18 hours.

Note 5 - From and after the date that safety valve position from the acoustic monitor is unavailable, reactor operation may continue provided safety valve position can be determined from recorder 2-166 (thermocouple, 0-600°F) and recorder 16-19-44 (drywell pressure 0-80 psia). If both indications are not available, the reactor shall be in a hot shutdown condition in six hours and in a cold shutdown condition in the following 18 hours.

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TABLE 4.7.6

CALIBRATION FREQUENCIES

POST-ACCIDENT INSTRUMENTATION

Parameter	Calibration	Instrument Check
Drywell Atmosphere Temperature	every 6 months	once each day
Drywell and Torus Pressure	every 6 months	once each day
Torus Water Level	every 6 months	once each shift
Torus Water Temperature	every 6 months	once each day
Reactor Pressure	every 6 months	once each day
Reactor Vessel Water Level	every 6 months	once each day
Control Rod Position	(note 5)	once each day
Neutron Monitor	Same as reactor protection systems	once each day
Torus Air Temperature	every 6 months	once each day
Safety/Relief Valve Position	every refueling outage (Note 9) (a Functional Test to be performed quarterly)	once each day
Safety Valve Position	every refueling outage (Note 9) (a Functional Test to be performed quarterly)	once each day

TABLE 4.2 NOTES

1. Initially once per month; thereafter, a longer interval as determined by test results on this type of instrumentation.

2. During each refueling outage, simulated automatic actuation which opens all pilot valves shall be performed such that each trip system logic can be verified independent of its redundant counterpart.

3. Trip system logic calibration shall include only time delay relays and timers necessary for proper functioning of the trip system.

4. This instrumentation is excepted from functional test definition. The functional test will consist of injecting a simulated electrical signal into the measurement channel.

5. Check control rod position indication while performing the surveillance requirement of section 3.3.

6. Functional tests, calibrations and instrument checks are not required when these instruments are not to be operable or tripped. Functional tests shall be performed before each startup with a required frequency not to exceed once per week. Calibration shall be performed prior to or during each startup or controlled shutdowns with a required frequency not to exceed once per week. Instrument checks shall be performed at least once per day during those periods when instruments are required to be operable.

7. This instrumentation is excepted from the functional test definitions and shall be calibrated using simulated electrical signals once every three months.

8. Functional tests and calibrations are not required when systems are not required to be operable.

9. The thermocouples associated with Safety/Relief Valves and Safety Valve Position, that may be used for backup position indication, shall be verified to be operable every operating cycle.

5. The Plant Health Physicist shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1 (September 1975).

6. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

E. A Fire Brigade of at least 5 members shall be maintained onsite at all times. // This excludes 2 members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency.

//Fire Brigade composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.

TABLE 6.1.1

Vermont Yankee staff positions that shall be filled by personnel holding Senior Operator and Operator licenses are indicated in the following table:

Title	License
Operations Supervisor	Senior Operator
Shift Supervisor	Senior Operator
Supervisory Control Room Operator	Operator
Control Room Operator	Operator

MINIMUM SHIFT CREW PERSONNEL & LICENSE REQUIREMENTS	Conditions		
	Normal Operation	Plant Startup	Cold Shutdown
Shift Supervisor	(1)	(1)	-
Supervisory Control Room Operator	(1)	(1)	(1)
Control Room Operator	(1)	(1)	(1)
Auxiliary Operator	(2)	(2)	(1)
Shift Technical Advisor	(1)	(1)	-
Senior Operators License	(1)	(1)	(1)
Operators License	(2)	(2)	(1)

6.9 ENVIRONMENTAL QUALIFICATION

A. By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class 1E Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of License DPR-28 dated October 24, 1980.

B. By no later than December 1, 1980, complete and auditible records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise qualified.

6.10 INTEGRITY OF SYSTEMS OUTSIDE CONTAINMENT

A program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels will be implemented. This program shall include the following:

1) Provisions establishing preventive maintenance and periodic visual inspection requirements, and

2) System leakage inspections, to the extent permitted by system design and radiological conditions, for each system at a frequency not to exceed refueling cycle intervals. The systems subject to this testing are (1) Residual Heat Removal, (2) Core Spray, (3) Reactor Water Cleanup, (4) HPCI, (5) RCIC, and (6) sample systems.

6.11 IODINE MONITORING

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas* under accident conditions will be implemented. This program shall include the following:

1) Training of personnel,

2) Procedures for monitoring, and

3) Provisions for maintenance of sampling and analysis equipment.

* Areas requiring personnel access for establishing hot shutdown condition.