#### ATTACHMENT I

PROPOSED TECHNICAL SPECIFICATIONS CHANGES

RELATED TO

CORE EXPOSURE

POWER AUTHORITY OF THE STATE OF NEW YORK JAMES A. FITZPATRICK NUCLEAR POWER PLANT DOCKET NO. 50-333

MARCH 9, 1981

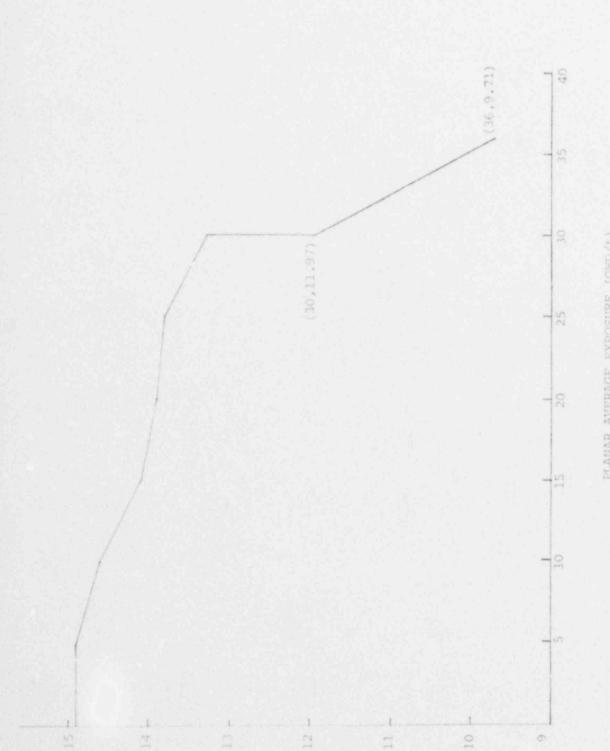


FIGURE 3.5-1 MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (MAPLHGR) VERSUS FLANAR AVERAGE EXPOSURE REFERENCE NEDO-21662-2 TABLE SA FULL CORE DRILLED INITIAL CORE

LINEAR HEAT GENERATION RATE (kw/ft)

FIGURE 3.5-2 MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (MAPLHCR) VERSUS PLANAR AVERAGE EXPOSURE (36,9,23) REFERENCE NEDO-21662-2 TABLE 58 PLANAR AVERAGE EXPOSURE (GWD/t) INITIAL CORE TYPE FULL CORE DRILLED 101 9

MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE (kw/ft)

ATTACHMENT II

SAFETY EVALUATION OF

CHANGES RELATED TO

CORE EXPOSURE

POWER AUTHORITY OF THE STATE OF NEW YORK JAMES A. FITZPATRICK NUCLEAR POWER PLANT DOCKET NO. 50-333

MARCH 9, 1981

#### Section I - Description of Modification

The proposed modification extends MPLHGR limits of 7x7 fuel to 36 GWD/t, thus extending the present limits of 30 GWD/t. The affected Technical Specification figures are Figures 3.5-1 and 3.5-2 and data for extending them were calculated and supplied by General Electric by letter dated October 21, 1980 (attached). These new data constitute a revision to the MAPLHGR Tables 5A and 5B in General Electric Report NEDO-21662-2. The General Electric data was modified for this application to incorporate penalties of 10% at 30 GWD/t and 20% at 36 GWD/t, as agreed in a telephone conversation with NRC on March 9, 1981.

#### Section II - Purpose of the Modification

During the present operating cycle (Cycle 4) the peak nodal exposure for 7x7 fuel is expected to surpass 30 GWD/t. This modification provides MAPLGHR limits consistent with presently accepted ECCS analysis, which allow operation beyond the present limit of 30 GWD/t.

#### Section III - Impact of the Change

The revised extension in MAPLHGR limits are more conservative than the attached amendment to General Electric report NEDO-21662-2, "Loss-of-Coolant Accident Analysis Report for James A. FitzPatrick Nuclear Power Plant" (lead plant) which was submitted to the Commission in a letter from George T. Berry to Robert W. Reid, Chief of Operating Reactors Branch, on July 29, 1977. This report forms the basis of the presently accepted ECCS analysis for the FitzPatrick plant; an SER issued by the Commission on September 16, 1977 references the Power Authority submittal. Thus the proposed modifications will not alter the conclusions reached in the FSAR and SER accident analysis.

### Section IV - Implementation of the Modification

The modification as proposed will not impact the ALARA or Fire Protection Program at JAF.

#### Section V - Conclusion

The incorporation of these modifications: a) will not increase the probability nor the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the Safety Analysis Report; b) will not increase the possibility for an accident or malfunction of a different type than any evaluated previously in the Safety Analysis Report; and c) will not reduce the margin of safety as defined in the basis for any Technical Specification, and d) does not constitute an unreviewed safety question.

#### Section VI - References

- (a) JAF FSAK
- (b) JAF SER
- (c) GE report NEDO-21662-2, "Loss-of-Coolant Accident Analysis Report for James A. FitzPatrick Nuclear Power Plant"

### GENERAL & ELECTRIC

GENERAL ELECTRIC COMPANY, 175 CURTNER AVE., SAN JOSE, CALIFORNIA 95125

NUCLEAR FUEL

AND SERVICES

DIVISION

October 21, 1980 NEX: 80-217

Mr. J. M. Clabby Power Authority State of New York 10 Columbus Circle New York, NY 10019

Subject: Errata and Addenda Sheet No. 2 - LOCA Analysis Report

for James A. FitzPatrick Nuclear Power Plant (Lead Plant)

Dear Jim:

Enclosed for your information and use are sixty-five (65) copies of the subject Errata and Addenda Sheet. The purpose of this change is the addition of MAPLHGR limits at 35,000 and 40,000 MWd/ST for FitzPatrick Initial Core Fuel types 2 and 3.

Very truly yours,

C. M. Richards, Manager

Fuel Projects - Northeast Region

M/C 193 - (408) 925-6509

mq Enclosures

cc: JD Leaser, w/att. (1)

KI May

-CS Orogvany, w/att. (1)

G. Rorke, w/att. (1)

RECEIVED

007 31 1980

FOY EN AUTHORITY

JAJR PLANT

## GENERAL @ ELECTRIC

PUBLICATION T.I.E NO	
TITLE LOC	A ANALYSIS REPORT FOR A. FITZPATRICK NUCLEAR
	PLANT (LEAD PLANT)
ISSUE DATE	July 1977

# ERRATA And ADDENDA

NO _	2		
DATE _	October 1980		
NOTE	Correct all copies of the applicable publication as specified below.		

REFERENCES (SECTION PAGE PARAGRAPH, LINE)		INSTRUCTIONS (CORRECTIONS AND ADDITIONS)		
01	Page 4-7	Replace with new page 4-7.		
		*		

Table 5A
MAPLHGR VERSUS AVERAGE PLANAR EXPOSURE

Plant: FitzPatrick		Fuel Type: Initial Core Type 2		
Average Planar Exposure (MWd/t)	MAPLHGR (kW/ft)	PCT (°F)	Oxidation Fraction	
200	14.9	2198	0.032	
1000	14.9	2196	0.031	
5000	14.9	2198	0.030	
10000	14.6	2199	0.031	
15000	14.1	2197	0.075	
20000	13.9	2197	0.075	
25000	13.8	2198	0.071	
30000	13.3	2189	0.075	1
35000	12.4	2065	0.050	
40000	11.1	1870	0.025	7

Table 5B
MAPLHGR VERSUS AVERAGE PLANAR EXPOSURE

Flant: FitzPatrick		Fuel Type:	Initial Core Type 3	
Average Planar		1		
Exposure (MWd/t)	MAPLHGR (kW/ft)	PCT (°F)	Oxidation Fraction	
200	14.7	2197	0.032	
1000	14.8	2197	0.032	
5000	15.0	2197	0.032	
10000	14.6	2198	0.030	
15000	14.1	2199	0.077	
20000	13.9	2196	0.077	
25000	13.7	2199	0.079	
30000	13.1	2198	0.050	
35000	11.8	2111	0.068	
40000	10.5	1933	0.035	