

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No. 50-266/81-05; 50-301/81-04

Docket No. 50-266; 50-301

License No. DRP-24; DPR-27

Licensee: Wisconsin Electric Power Company
231 West Michigan
Milwaukee, WI 53203

Facility Name: Point Beach Nuclear Power Plant, Units 1 & 2

Inspection At: Point Beach Site, Two Creeks, WI

Inspection Conducted: March 2-31, 1981

Inspectors: *J. D. Smith for*
W. G. Guldemon

4-16-81

J. L. Smith for
R. L. Hague

4-16-81

Approved By: *J. E. Konklin*
J. E. Konklin, Acting Chief
Projects Section 2A

4/16/81

Inspection Summary

Inspection on March 2-6, 9-13, 16-20, 23-27, 30-31, 1981 (Report No. 50-266/81-05; 50-301/81-04)

Areas Inspected: Routine resident inspection of Operational Safety Verification, Monthly Maintenance Observation, Monthly Surveillance Observation, Followup on Licensee Event Reports, IE Bulletin and Circular Followup, Review of Plant Observations, Design, Design Changes, and Modifications, Review of Periodic and Special Reports, Independent Inspection Effort, TMI Action Plan Items, Followup on Regional Requests. The inspection involved a total of 200 inspector-hours onsite by two inspectors including 46 inspector-hours on off-shifts.

Results: No items of noncompliance were identified.

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DETAILS

1. Persons Contacted

*G. A. Reed, Manager, Nuclear Power Division
J. J. Zach, Superintendent Technical Services
T. J. Koehler, Operations Superintendent
J. C. Reisenbuechler, I&C Engineer
W. J. Herrman, Maintenance Superintendent
R. S. Bredvad, Health Physicist
*R. E. Link, Assistant to the Manager
*F. A. Zeman, Office Supervisor

The inspectors also talked with and interviewed members of the Operations, Maintenance, Health Physics, and Instrument and Control Sections.

*Denotes personnel attending exit interviews.

2. Operational Safety Verification

The inspector observed control room operations, reviewed applicable logs and conducted discussions with control room operators during the month of March. The inspector verified the operability of selected emergency systems, reviewed tagout records and verified proper return to service of affected components. Tours of both auxiliary buildings and turbine buildings were conducted to observe plant equipment conditions, including potential fire hazards, fluid leaks, and excessive vibrations and to verify that maintenance requests had been initiated for equipment in need of maintenance. The inspector by observation and direct interview verified that the physical security plan was being implemented in accordance with the station security plan.

The inspector observed plant housekeeping/cleanliness conditions and verified implementation of radiation protection controls. During the month of March, the inspector walked down the accessible portions of the safety injection, auxiliary feed, emergency diesel generating systems and containment spray systems to verify operability. The inspector also witnessed portions of the radioactive waste system controls associated with radwaste shipments and barreling. This included a confirmatory radiation measurement of a radwaste shipment and attendance at the briefing of the truck driver.

These reviews and observations were conducted to verify that facility operations were in conformance with the requirements established under technical specifications, 10 CFR, and administrative procedures.

No items of noncompliance or deviations were identified.

3. Monthly Maintenance Observation

Station maintenance activities of safety related systems and components listed below were observed/reviewed to ascertain that they were conducted in accordance with approved procedures, regulatory guides and industry codes or standards and in conformance with technical specifications.

The following items were considered during this review: the limiting conditions for operation were met while components or systems were removed from service; approvals were obtained prior to initiating the work; activities were accomplished using approved procedures and were inspected as applicable; functional testing and/or calibrations were performed prior to returning components or systems to service; quality control records were maintained; activities were accomplished by qualified personnel; parts and materials used were properly certified; radiological controls were implemented; and, fire prevention controls were implemented.

Work requests were reviewed to determine status of outstanding jobs and to assure that priority is assigned to safety related equipment maintenance which may affect system performance.

The following maintenance activities were observed/reviewed:

Replacement of R-15 Radiation Detector

Following completion of maintenance on the 4D diesel generator (relay replacement), the inspector verified that these systems had been returned to service properly.

No items of noncompliance or deviations were identified.

4. Monthly Surveillance Observation

The inspector observed technical specifications required surveillance testing on the REI 6.0; Quarterly Flux Mapping for Unit 1 and verified that testing was performed in accordance with adequate procedures, that test instrumentation was calibrated, that limiting conditions for operation were met, that removal and restoration of the affected components were accomplished, that test results conformed with technical specifications and procedure requirements and were reviewed by personnel other than the individual directing the test, and that any deficiencies identified during the testing were properly reviewed and resolved by appropriate management personnel.

The inspector also witnessed portions of the following test activities:

PTS 2 Redundant Systems Test on 4D diesel generator

No items of noncompliance or deviations were identified.

5. Licensee Event Reports Followup

All Licensee Event Reports received to date have been closed out in previous inspection reports.

6. IE Bulletin Followup

For the IE Bulletin listed below the inspector verified that the written response was within the time period stated in the bulletin, that the written response included the information required to be reported, that the written response included adequate corrective action commitments based on information presentation in the bulletin and the licensee's response, that licensee management forwarded copies of the written response to the appropriate onsite management representatives, that information discussed in the licensee's written response was accurate, and that corrective action taken by the licensee was as described in the written response.

80-06 Engineered Safety Feature Reset Controls

No items of noncompliance or deviations were identified.

7. IE Circular Followup

All outstanding circulars were reviewed. It was determined that none could be closed out during the reporting period.

8. Review of Plant Operations

During the month of March the inspector reviewed the following activities:

a. Training

The inspector attended one of the licensee's operator qualification lectures on emergency planning and verified that lesson plan objectives were met.

b. Security

The inspector observed that three individuals achieved acceptable scores during the conduct of weapons and physical fitness tests.

c. Licensee Action Concerning Identified Problems

The inspector reviewed corrective actions taken by the licensee pertaining to recurring failures and resolution of identified discrepancies involving safety-related components.

No items of noncompliance or deviations were identified.

9. Plant Trips

Following the Unit 1 trip on March 31, 1981 at 0051 the inspector ascertained the status of the reactor and safety systems by observation of control room indicators and discussions with licensee personnel concerning plant parameters, emergency system status and reactor coolant chemistry. The inspector verified the establishment of proper communications and reviewed the corrective actions taken by the licensee.

All systems responded as expected, and the plant was returned to operation on March 31, 1981 at 0940.

No items of noncompliance or deviations were identified.

10. Design, Design Changes, and Modifications

The inspectors reviewed selected modifications not requiring approval by the Office of Nuclear Reactor Regulation to verify that the changes were reviewed and approved in accordance with 10 CFR 50.59. Also verified were that the design changes were reviewed and approved in accordance with Technical Specifications and established QC/QA procedures, that the changes were controlled by established procedures, that tests of the changes were conducted and the results reviewed, and that drawings and procedures were changed as appropriate.

The following modifications were reviewed:

- a. Reset Circuitry Modification Safeguards
- b. Fire Protection - Install Hose Reels Inside Containment

For the Reset Circuitry Modification, changes to maintenance procedures, drawings, and logs had not been documented in the modification package. This was pointed out to the licensee who took immediate action to have the required documentation made.

No items of noncompliance or deviations were identified.

11. Review of Periodic and Special Reports

The inspectors reviewed the licensee's "Annual Results and Data Report 1980". The review was conducted to ascertain whether the information reported by the licensee was technically adequate and satisfied the applicable reporting requirements established in 10 CFR, the facility license, and Technical Specifications. Included in the report were highlights of operation of both Unit 1 and Unit 2 for 1980, facility changes, tests and experiments, number of personnel and man-rem exposure by work and job function, and steam generator tube inservice inspection results.

No items of noncompliance or deviations were identified.

12. Independent Inspection Effort

During the course of the inspection period the inspectors reviewed licensee activities in the following areas to ascertain that they were conducted in accordance with regulatory requirements and the Technical Specifications: changes to procedures, qualification of Duty and Call Technical Advisors and radiological surveys.

The inspectors reviewed all procedure changes, both temporary and permanent, for the period January 1, 1980 to February 1, 1981 to verify that proper approvals and reviews had been applied to the changes and that proper documentation of the changes had been retained. A sampling of temporary changes to major procedures were reviewed to determine technical adequacy and justification of the changes. Based on these reviews; it was determined that the licensee's program for controlling procedure changes is satisfactory. In two instances temporary changes to major procedures were not reviewed by the Manager's Supervisory Staff within the required two weeks. These instances were discovered by the licensee and instructions were promulgated to all station personnel involved in procedure changes reminding them of the required timeliness for approval of procedural changes. To further control procedure changes the licensee has instituted the use of a new procedure change form which documents the required reviews and safety evaluations.

A review of the licensee's Duty and Call Technical Advisor training program and records indicates, as stated in their February 26, 1981 letter from C. W. Fay to H. R. Denton, that all assigned personnel have completed the training program as described in the letter with the exception of a 40 hour course on mitigating core damage. Some material on this subject has been incorporated in their training. However, a more extensive 40 hour course is being arranged to be taught onsite by Westinghouse. The exact date of completion is dependent on scheduling this course.

During a routine tour of the controlled side of the facility, the inspector noted through independent surveys of indicated "hot spots" that most readings obtained were significantly less than the posted dose rates. While overposting is not a significant problem, the inspector did question the method of establishing "hot spots" and if any program was in effect to periodically re-evaluate them.

Some of the postings which indicated up to ten times the measured dose rate were nearly a year old. In discussions with the Health Physicist, it was determined that no routine program for re-evaluation of "hot spots" exists. However, the licensee did commit to inclusion of "hot spot" surveys as part of the weekly routine survey. Administrative forms have been prepared to this end.

13. TMI Action Plan Inspection Requirements

The inspectors reviewed licensee actions in accordance with Revision 1 of Temporary Instructions 2515/42, 43, and 44. The attached tables present the current status of implementation of TMI Action Plan requirements called for by the referenced temporary instructions.

No items of noncompliance or deviations were identified.

14. Followup on Regional Requests

On March 23, 1981, the inspectors received a memorandum from Region III requesting data to support completion of Manual Chapter 61727B on RCS leakage. This information was forwarded to Region III by memorandum dated March 23, 1981.

On March 23, 1981, the inspectors received a request from Region III to update the computer tracking system for all TMI Action Plan items. This information was forwarded to Region III on March 24, 1981.

No items of noncompliance or deviations were identified.

15. Exit Interview

The inspector met with licensee representatives (denoted in Paragraph 1) throughout the month and at the conclusion of the inspection period and summarized the scope and findings of the inspection activities. The licensee acknowledged these findings.

TMI ACTION PLAN STATUS
PROCEDURES AND STAFFING

TASK	SUBJECT	STATUS
I.A.1.1	STA INTERIM STAFFING	COMPLETE. STA's HAVE BEEN ON DUTY SINCE 1/1/80 IN ACCORDANCE WITH A S. BURSTEIN (WE) TO H. R. DENTON (NRC) LETTER DATED 12/31/79.
I.A.1.1	STA TRAINED PER LL CAT "B"	ALL TRAINING EXCEPT THAT REQUIRED FOR MITIGATING CORE DAMAGE WAS COMPLETE AS OF 3/1/81. THE LICENSEE IS STILL ATTEMPTING TO ESTABLISH A TRAINING PROGRAM FOR MITIGATING CORE DAMAGE. PER A 12/23/80 LETTER FROM C. W. FAY (WE) TO H. R. DENTON (NRC) THIS REQUIREMENT WILL BE SATISFIED NO LATER THAN 10/1/81.
I.A.1.2	SHIFT SUPERVISOR RESPONSIBILITIES	COMPLETE
I.A.1.3	SHIFT MANNING	COMPLETE. THE PROCEDURES ARE CONTAINED IN THE POINT BEACH NUCLEAR PLANT ADMINISTRATIVE POLICIES AND PROCEDURES MANUAL SECTION 4.3.3 DATED 1/5/81
I.A.2.1	MODIFY TRAINING	COMPLETE. THE TRAINING PROGRAM WAS SUBMITTED IN A 8/7/80 LETTER FROM C. W. FAY (WE) TO H. R. DENTON (NRC).
I.C.1	SHORT TERM ACCIDENT AND PROCEDURE REVIEW (ITEM 1)	COMPLETE
I.C.1	SHORT TERM ACCIDENT AND PROCEDURE REVIEW (ITEMS 2b & 3b)	THESE ITEMS ARE PENDING REVIEW OF THE WESTINGHOUSE OWNERS GROUP SUBMITTAL TO THE NRC.
I.C.2	SHIFT RELIEF/TURNOVER	COMPLETE
I.C.3	SHIFT SUPERVISOR RESPONSIBILITIES	COMPLETE
I.C.4	CONTROL ROOM ACCESS	COMPLETE
I.C.5	PROCEDURE FOR OPERATING EXPERIENCE FEEDBACK	COMPLETE. THE PROCEDURES ARE CONTAINED IN THE POINT BEACH NUCLEAR PLANT ADMINISTRATIVE POLICIES AND PROCEDURES MANUAL SECTION 3.15.7 DATED 12/19/80
I.C.6	VERIFICATION OF OPERATING ACTIVITIES	COMPLETE. THE PROCEDURES ARE CONTAINED IN THE POINT BEACH NUCLEAR PLANT ADMINISTRATIVE POLICIES AND PROCEDURES MANUAL SECTION 4.14 DATED 6/20/80

TMI ACTION PLAN STATUS

HARDWARE

TASK	SUBJECT	STATUS
II.B.1	RCS VENTS	PER NUREG 0737 THIS ITEM IS COMPLETE THROUGH THE DESIGN PHASE
II.B.2	PLANT SHIELDING	THE DESIGN REVIEWS HAVE BEEN COMPLETED USING THE SOURCE TERMS PROVIDED IN NUREG 0578. HOWEVER, NUREG 0737 CHANGED THE SOURCE TERM ASSUMPTIONS AND THE LICENSEE IS EVALUATING THE IMPACT ON THE PREVIOUS REVIEWS. PER A 12/23/80 LETTER FROM C. W. FAY (WE) TO H. R. DENTON (NRC) THE LICENSEE HAS COMMITTED TO COMPLETE PLANT MODIFICATIONS BY 6/1/82 SUBJECT TO EQUIPMENT AVAILABILITY.
II.D.3	DIRECT INDICATION OF RELIEF AND SAFETY VALVE POSITIONS	COMPLETE. TECHNICAL SPECIFICATIONS WERE SUBMITTED IN FEBRUARY, 1981.
II.E.1.2	AUXILIARY FEEDWATER SYSTEM AUTOMATIC INITIATION (ITEM 1a)	COMPLETE. ORIGINAL PLANT DESIGN.
II.E.1.2	AUXILIARY FEEDWATER SYSTEM AUTOMATIC INITIATION (ITEM 1b)	COMPLETE. ORIGINAL PLANT DESIGN
II.E.1.2	AUXILIARY FEEDWATER SYSTEM FLOW INDICATION	THE PRESENT SYSTEM CONSISTS OF A COMBINATION OF STEAM GENERATOR LEVEL INDICATION AND AUXILIARY FEED PUMP DISCHARGE FLOW INDICATION. THE PUMP DISCHARGE FLOW INDICATION IS BEING UPGRADED TO SAFETY GRADE. THE SYSTEM IS PRESENTLY BEING MODIFIED TO INCLUDE SAFETY GRADE INDICATION OF FLOW TO EACH STEAM GENERATOR. THESE MODIFICATIONS ARE TO BE COMPLETE BY 7/1/81 PER A 12/23/80 LETTER FROM C. W. FAY (WE) TO H. R. DENTON (NRC). TECHNICAL SPECIFICATIONS COVERING THE PRESENTLY INSTALLED SYSTEM WERE SUBMITTED IN FEBRUARY, 1981.
II.E.3.1	EMERGENCY POWER FOR PRESSURIZER HEATERS	COMPLETE. TECHNICAL SPECIFICATIONS WERE SUBMITTED ON 2/4/81.
II.E.4.1	DEDICATED HYDROGEN PENETRATIONS	COMPLETE. ORIGINAL PLANT DESIGN.
II.E.4.2	CONTAINMENT ISOLATION DEPENDABILITY	COMPLETE PER NUREG 0737. THE LICENSEE IS ADDING CONTAINMENT ISOLATION VALVES ON THE SEAL WATER RETURN AND AUXILIARY CHARGING LINES. THE LICENSEE HAS TAKEN EXCEPTION TO THE GUIDELINES ON THE CONTAINMENT PRESSURE SETPOINT. THEY CONTEND THAT 1 PSI ABOVE THE NORMAL OPERATING PRESSURE MAY BE INSUFFICIENT TO PRECLUDE INADVERTENT AND UNWANTED CONTAINMENT ISOLATIONS. HOWEVER, THE LICENSEE, IN A 12/23/80 LETTER FROM C. W. FAY (WE) TO H. R. DENTON (NRC) HAS COMMITTED TO EVALUATE THE SITUATION. NO DATE FOR COMPLETING THIS EVALUATION WAS PROVIDED.

TMI ACTION PLAN STATUS
PROCEDURES AND STAFFING

TASK	SUBJECT	STATUS
II.B.4	TRAINING FOR MITIGATING CORE DAMAGE	THE LICENSEE HAS YET TO FINALIZE PLANS FOR THIS TRAINING. THIS IS EXPECTED TO HAPPEN AFTER 4/1/81. PER A 12/23/80 LETTER FROM C. W. FAY (WE) TO H. R. DENTON (NRC) THE TRAINING WILL BE COMPLETED BEFORE 10/1/81.
II.E.4.2	CONTAINMENT ISOLATION DEPENDABILITY	COMPLETE. THE CONTAINMENT PURGE SUPPLY AND EXHAUST VALVES ARE ADMINISTRATIVELY SHUT (BY RED TAGGING THE CONTROL SWITCHES) EXCEPT UNDER COLD SHUTDOWN CONDITIONS.
II.F.2	IDENTIFICATION OF AND RECOVERY FROM CONDITIONS LEADING TO INADEQUATE CORE COOLING	COMPLETE
II.K.3	B&O TASK FORCE ITEM 22	N/A, BWR ONLY
III.D.1.1	PRIMARY COOLANT OUTSIDE CONTAINMENT	COMPLETE. THE PROGRAM HAS BEEN IMPLEMENTED IN ACCORDANCE WITH A 3/14/80 LETTER FROM C. W. FAY (WE) TO H. R. DENTON (NRC). THE LICENSEE IS CURRENTLY CHANGING TO A YEARLY TESTING SCHEDULE FOR BOTH UNITS COINCIDENT WITH REFUELING OUTAGES.

TMI ACTION PLAN STATUS

HARDWARE

TASK	SUBJECT	STATUS
II.F.1	ADDITIONAL ACCIDENT MONITORING INSTRUMENTATION (ITEMS 4, 5, & 6)	THE LICENSEE HAS COMMITTED TO A 1/1/82 COMPLETION DATE FOR ITEMS 4 & 5 CONTINGENT ON EQUIPMENT AVAILABILITY. THIS COMMITMENT WAS MADE IN A 12/23/80 LETTER FROM C. W. FAY (WE) TO H. R. DENTON (NRC). THIS SAME LETTER STATES SINCE THERE IS NO QUALIFIED HYDROGEN DETECTION EQUIPMENT AVAILABLE, THAT NO COMMITMENT CAN BE MADE TO ITEM 6.
II.G.1	POWER SUPPLIES FOR PRESSURIZER RELIEF VALVES, BLOCK VALVES AND LEVEL INDICATION (ITEM 1)	COMPLETE. TECHNICAL SPECIFICATIONS WERE SUBMITTED ON 2/4/81.
II.K.3	B&O TASK FORCE ITEM 9	COMPLETE. PER 6/11/80 LETTER FROM C. W. FAY (WE) TO H. R. DENTON (NRC).

TMI ACTION PLAN STATUS

HEALTH PHYSICS/EMERGENCY PLANNING

TASK	SUBJECT	STATUS
II.B.3	POST-ACCIDENT SAMPLING (ITEM 1)	THE LICENSEE HAS PROCURED THE EQUIPMENT AND MADE IT OPERATIONAL FOR AN INTERIM SYSTEM. PROCEDURES FOR THE REQUIRED SAMPLES ARE CONTAINED IN THE POINT BEACH NUCLEAR PLANT HEALTH PHYSICS ADMINISTRATIVE CONTROL POLICIES AND PROCEDURES MANUAL SECTIONS 17.6.1, 17.6.2, 17.6.3, 17.6.4, AND 17.6.5. THE LICENSEE HAS TAKEN EXCEPTION TO THE REQUIREMENT FOR CHLORIDE ANALYSIS BASED ON THEIR CONTENTION THAT THERE IS NO CHLORIDE SOURCE FOR THE RCS AND THE DIFFICULTY OF THE ANALYSIS.
II.F.1	INSTRUMENTATION FOR MONITORING ACCIDENT CONDITIONS	THE LICENSEE HAS ESTABLISHED INTERIM CAPABILITY. PROCEDURES FOR THE REQUIRED DETERMINATIONS ARE CONTAINED IN THE POINT BEACH NUCLEAR PLANT HEALTH PHYSICS ADMINISTRATIVE CONTROL POLICIES AND PROCEDURES MANUAL SECTIONS 17.6.4 and 17.6.5.
III.A.1.2	UPGRADE EMERGENCY SUPPORT FACILITIES - SHORT TERM; TECHNICAL SUPPORT CENTER AND OPERATIONAL SUPPORT CENTER	COMPLETE. THE TEMPORARY TECHNICAL SUPPORT CENTER IS THE CONFERENCE ROOM IN THE ADMINISTRATIVE BUILDING. PLANT MONITORING INSTRUMENTATION WAS MADE OPERATIONAL IN EARLY MARCH, 1981. CONSTRUCTION IS UNDERWAY ON THE NEW PERMANENT TECHNICAL SUPPORT CENTER. THE ENERGY INFORMATION CENTER IS CURRENTLY DESIGNATED AS THE OPERATIONAL SUPPORT CENTER.
II.A.1.2	UPGRADE EMERGENCY SUPPORT FACILITIES - SHORT TERM; EOF	COMPLETE. AN INTERIM EOF HAS BEEN ESTABLISHED IN THE ENERGY INFORMATION CENTER APPROXIMATELY 200 YARDS FROM THE PLANT.
III.D.3.3	INPLANT RADIATION MONITORING (ITEM 1)	COMPLETE. SATISFACTORY INTERIM PROCEDURES ARE IN PLACE AND THE SUPPORTING EQUIPMENT IS OPERATIONAL. ADDITIONAL UPGRADING IS IN PROGRESS. A RECIPROCAL AGREEMENT EXISTS WITH THE KEWAUNEE NUCLEAR POWER PLANT FOR USE OF LOW BACKGROUND LABORATORY FACILITIES.
III.D.3.3	INPLANT RADIATION MONITORING (ITEM 2)	SEE III.D.3.3 ABOVE.