



LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION

P.O. BOX 618, NORTH COUNTRY ROAD • WADING RIVER, N.Y. 11792

SNRC-563

May 15, 1981



Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

SHOREHAM NUCLEAR POWER STATION - UNIT 1
DOCKET NO. 50-322

Dear Mr. Denton:

Forwarded herewith are fifteen (15) copies of our positions related to Post-TMI Requirements outlined in NUREG-0737, "Clarification of TMI Action Plan Requirements". The twenty-one specific items addressed are listed in Attachment 1 to this letter. Where applicable, our responses to certain NUREG-0578 items (SNRC-503, dated 8/29/80) are superceded by the respective responses contained herein.

This submittal is the second of three submittals dealing with NUREG-0737. The first submittal contained seventeen specific items and was forwarded via letter SNRC-557 dated 4/15/81. The last submittal is scheduled for 5/30/81 and will contain the remaining NUREG-0737 items as listed in Attachment 2.

Very truly yours,

J. P. Novarro
Project Manager
Shoreham Nuclear Power Station

RWG:mc
Attachment
cc: J. Higgins

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ATTACHMENT 1NUREG-0737 ITEMS INCLUDED IN THIS SUBMITTAL

I.A.1.2	Shift Supervisor
I.A.1.3	Shift Manning
I.A.3.1	Revise Scope and Criteria for Licensing Examinations
I.C.3	Shift Supervisor Responsibility
I.C.7	NSSS Vendor Review of Procedures
I.C.8	Pilot Monitoring of Selected Emergency Procedures
I.D.1	Control Room Design Reviews
II.B.2	Plant Shielding
II.B.3	Post-Accident Sampling
II.F.1	Additional Accident Monitoring Instrumentation
II.K.1.10	Operability Status
II.K.1.22	Auxiliary Heat Removal System Procedures
II.K.1.23	Reactor Vessel Level Indication
II.K.3.3	Reporting Safety Relief Valve Failures
II.K.3.13	HPCI and RCIC Initiation Levels
II.K.3.16	Challenges and Failures to Relief Valves
II.K.3.18	Modification of ADS Logic
II.K.3.24	Space Cooling for HPCI and RCIC
II.K.3.25	Power on Pump Seals
III.D.3.3	In-Plant I ₂ Radiation Monitoring
III.D.3.4	Control Room Habitability

ATTACHMENT 2NUREG-0737 ITEMS SCHEDULED FOR SUBMITTAL ON 5/30/81

I.A.2.1	Immediate Upgrading of RO & SRO Training and Qualification
I.B.1.2	Evaluation of Organization and Management
I.C.1	Short Term Accident and Practice Review
I.C.6	Verify Correct Performance of Operating Activities
I.D.2	Plant Safety Parameter Display Console
I.G.1	Training During Low Power Testing
II.B.1	Reactor Coolant System Vents
II.B.4	Training for Mitigating Core Damage
II.D.1	Relief and Safety Valve Testing Requirements
II.E.4.2	Containment Isolation Dependability
II.F.2	Instrumentation for Detection of Inadequate Core Cooling
II.F.3	Instrumentats for Monitoring Accident Conditions
II.K.3.30	Small break LOCA Loads
II.K.3.31	Plant-Specific Analysis
III.A.1.1	Emergency Preparedness Short Term
III.A.1.2	Upgrade Emergency (Based on Security Bldg.)
III.A.2	Emergency Preparedness
III.D.1.1	Primary Coolant Outside Containment