OPERATING DATA REPORT

DOCKET NO. 50-366
DATE 05-10-81
COMPLETED BY GERRY ELAM
TELEPHONE (912) 367-7781 x 203

OPERATING STATUS

1. Unit Name: E. I. Hatch Nuclear Plant Unit 2
2. Reporting Period: 04-81
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 817.0
5. Design Electrical Rating (Net MWe): 784.0
6. Maximum Dependable Capacity (Gross MWe): 805.7
7. Maximum Dependable Capacity (Net MWe): 772.7
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	719	2879	14496
12. Number Of Hours Reactor Was Critical	707.3	1431.0	9664.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	693.8	1301.5	8989.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1566557	2733083	19830981
17. Gross Electrical Energy Generated (MWH)	507360	888210	6555490
18. Net Electrical Energy Generated (MWH)	485381	839307	6241421
19. Unit Service Factor	96.5	45.2	62.0
20. Unit Availability Factor	96.5	45.2	62.0
21. Unit Capacity Factor (Using MDC Net)	87.4	37.7	55.7
22. Unit Capacity Factor (Using DER Net)	86.1	37.2	54.9
23. Unit Forced Outage Rate	3.5	22.5	13.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-366
DATE 05-10-81
COMPLETED BY GERRY ELAM
TELEPHONE (912) 367-7781 x 203

MONTH 04-81

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	750	17	699
2	742	18	725
3	737	19	732
4	6[illegible]	20	730
5	71[illegible]	21	601
6	745	22	729
7	743	23	731
8	741	24	731
9	470	25	734
10	361	26	708
11	62	27	736
12	624	28	735
13	706	29	731
14	[illegible]	30	729
15	696	31	
16	695		

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POOR ORIGINAL

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH 4-81

DOCKET NO. 50-366
UNIT NAME Hatch 2
DATE 5-10-81
COMPLETED BY G. H. Elam
TELEPHONE 912-367-7851

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
81-34	810404	F	5.0	A	5	NA	CH	HTEXCH	Load reduction due to loss of feedwater heaters
81-35	810409	F	20.0	B	5	NA	HC	HTEXCH	Load reduction to repair condenser tube leak
81-36	810410	F	14.2	B	3	NA	CE	InSTRU	Rx scram - Group 1 isolation while performing surveillance on low vacuum switches
81-37	810411	F	2.0	B	5	NA	CE	INSTRU	Recovery from above scram #81-36
81-38	810411	F	11.0	G	3	NA	EB	PUMPXX	Rx scram due to 2A RFP tripping which was caused by improper transfer of bus loads.
81-39	810411	F	18.0	G	5	NA	EB	PUMPXX	Recovery from above scram #81-38

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH 4-81

DOCKET NO. 50-366
UNIT NAME Hatch 2
DATE 5-10-81
COMPLETED BY G. H. Elam
TELEPHONE 912-367-7851

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
81-40	810416	S	5.0	F	5	NA	RB	CONROD	Load reduction to perform rod pattern adjustment
81-41	810421	F	10.5	B	5	NA	HC	HTEXCH	Load reduction to check for condenser tube leaks

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
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NARRATIVE REPORT
UNIT 2

April 10th	Rx auto scram at 1334 due to Group 1 isolation while test shop was performing surveillance on low vacuum switches per HNP-2-3104
April 10th	Rx critical at 2033 hrs, on line at 0344 hr. on April 11th
April 11th	Rx auto scram at 0621 due to low Rx water level caused when booster pump 2A tripped which in turn caused booster pump 2A to trip. This event happened during the transfer of house loads
April 11th	Rx critical at 1105 hrs., on line at 1722 hours

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR APRIL 1981

NUMBER	DATE COMPLETED	DESCRIPTION
80-4685	2-16-81	Performed PM (per HNP-2-6020) on main steam relief valves 2B21-F013A-H and K-M
80-4816	2-15-81	Repaired feedwater check valve 2B21-F010B after the valve failed LLRT
81-305	2-17-81	QC performed visual inspection of pipe supports and hangers 2B21-G001 per HNP-907
81-945	2-16-81	Repaired and calibrated SRV tailpipe pressure switches 2B21-N301A-M
80-4887	12-23-80	Moved valves 2B31-F029 and F030 to directly below the tee in the bottom head drain line (per DCR 80-345)
81-1643	3-15-81	Investigated overlap of IRM 'B' 2C51-K601B from range 1 to range 7
81-1571	3-12-81	Inspected all IRM penetrations cables and connections 2C51-K601A-H
81-304	3-7-81	QC performed visual inspection of pipe supports and hangers 2B21- 01 per HNP-907
81-1143	3-30-81	Tightened nut on bottom of pipe clamp 2E11-RSW-R40
80-5018	12-10-80	Removed pipe restraint 2E21-CS-H7, removed old paint, buffed, repainted and installed
81-725	2-11-81	Removed trip and throttle valves on RCIC turbine 2E51-C002, performed PT test and reinstalled
81-805	2-19-81	Modified hangers 2G11-RAd-H90 and 2G11-RAD-A91 and installed new hanger 2G11-RAD-HR700 per DCR 80-394

81-1049	2-19-81	Replaced damaged section of piping between drywell equipment sump to radwaste piping 2G11-F020 and 2G11-N011
80-2436	6-4-80	Replaced snubbers 2L31
81-736	2-16-81	Inspected and tested inaccessible safety related mechanical snubbers (2L31) per HNP-2-6804
81-1102	2-18-81	Replaced snubbers 2N37-TBP-R14A with new snubbers SN5330
80-1139	6-16-80	Made conduit penetrations and embedded sleeves per DCR 79-426 in radwaste building 2P65
81-1511	3-16-81	Removed strongback airlock door (2T23) for drywell entry and reinstalled after drywell closeout
81-1534	3-13-81	Removed S drywell equipment hatch 2T23 for MSIV work
81-124	2-11-81	2T43 reactor building Unit 2, pulled circuits 2TLX702C02 and TLXX702C04, removed old circuits 2MR2504, 2505, and 2506, removed smoke detector 2T43-43-N024B
81-755	2-10-81	Replaced air cylinders and air control valves on drywell to torus vacuum breakers 2T48-F323A-L
81-917	2-14-81	Replaced air supply line on torus to drywell vacuum breakers 2T48-F323 H&I
80-2930	10-21-80	Installed concrete pad, supported steel and chiller units for LPCI inverter cooling system 2Z41 per DCR 78-68
80-5101	12-18-80	Modified the 6" core drill (per DCR 78-55) to allow for clearance between the 4" fire protection pipe and block wall in cable spread 2Z43