

General Offices: 212 West Michigan Avenue, Jackson, Michigan 49201 • Area Code 517 788-0550

May 9, 1979

Director, Nuclear Reactor Regulation Att Mr Dennis L Ziemann, Chief Operating Reactors Branch No 2 US Nuclear Regulatory Commission Washington, DC 20555

DOCKET 50-155 - LICENSE DPR-6 BIG ROCK POINT PLANT - SCHEDULE FOR
SUBMITTAL OF ADDITIONAL INFORMATION
REGARDING FIRE PROTECTION MODIFICATIONS

NRC letter dated April 4, 1979 forwarded Amendment 25 to the Big Rock Point Technical Specifications. The NRC letter requested Consumers Power to provide a schedule for submitting additional information regarding certain plant modifications related to fire protection as discussed in the Safety Evaluation Report (SER) accompanying Amendment 25.

Based upon the best information currently available from our design contractor, the requested information will be submitted in accordance with the following schedule (modification numbers are the same as those used in the SER accompanying Amendment 25):

Modification	Information Requested	Submittal Schedule
3.1.1	Alternate Safe Shutdown System Design Description	December 31, 1979
3.1.3	Number, Type and Location of Fire Detection Devices	August 31, 1979
3.1.4	Location of Fire Pump Header Hose Connections and Isolation Valves	August 31, 1979
3.1.13	Cable Penetration Fire Stop Design Description	August 31, 1979
3.1.23	Location of Recirculation Pump Area Sprinkler Heads, Method of Actuation and System Design Density	August 31, 1979
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Modification	Information Requested	Submittal Schedule
3.1.25	Actions To Be Taken and Technical Specifications Changes Required To Remove Power From Shutdown Cooling Isolation Valves	Submitted by Letter Dated March 26, 1979
3.1.26	Reactor Depressurization System Insulation Design Description and Description of Areas To Be Protected	August 31, 1979

Roger W. Haston

for David P Hoffman Assistant Nuclear Licensing Administrator

CC JGKeppler, USNRC

DETAILS

1. Persons Contacted

- *J. Rang, Operations and Maintenance Superintendent
- *D. DeMoor, Technical Engineer
- *C. Axtell, Health Physicist
- *T. Brun, Chemical and Radiation Protection Supervi or
- G. Gilbody, QA Engineer

2. General

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This inspection, which began at approximately 3:30 p.m. on February 14, 1979, was conducted to examine radiation protection activities associated with the refueling outage that began on February 1, 1979. That afternoon and evening the inspector observed ongoing radiation protection work in containment and radiological conditions throughout the plant. A portable survey meter was used to confirm postings and otherwise examine radiation levels in the areas visited. Continuous radiation protection coverage was being provided for fuel inspection work at the fuel pool where levels ranging from 2 to 25 mR/hr were observed, and for inservice inspection work in the steam drum area where exposure rates approaching 400 mR/hr were inferred from timekeeping sheets used by the health physics technician. Checkout survey meters placed at strategic locations in containment and continuous air monitors were observed to be in good working order. Plant cleanliness and radiological conditions appeared satisfactory.

Protective clothing and respirator supplies at access control were adequate. The access control area had been enlarged and crowded conditions noted there in previous inspections were not evident during this inspection. A licensee representative stated that about 100 extra workers were expected onsite during the peak of the outage.

3. Radiation Protection Procedures

Outage radiation protection work is generally conducted under the normal radiation protection procedures. Radiation protection participates in outage planning and reviews procedures written by other groups for special outage work. The inspector reviewed a set of procedures written for removal of the ring sparger and confirmed that satisfactory cognizance was given to radiological protection requirements.