## LICENSEE EVENT REPORT

	CONTROL BLOCK: 1 1 1 1 6 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1	M I B R P 1 2 0 0 - 10 0 0 0 0 - 0 0 3 4 1 1 1 1 1 1 5 57 CAT 58
CON'T	SOURCE L 6 0 5 0 - 10 1 5 5 7 0 4 1 2 1 0 7 1 9 8 0 5 10 2 7 1 9 9
	During inspection following maintenance activity at 1500 hours, minor
0 2	
0 3	water leakage was observed at the interface between the reactor vessel and
0 4	F-2 rod drive housing. No leakage was noted at any other similar housing
0 5	interface. Shutdown condition was established pending evaluation. No
0 6	prior leakage at housing has been observed. No hazard to the public
0 7	occurred. Reportable per Tech. Spec. 6.9.2.a.(3)
7 8	9 SYSTEM CAUSE CAUSE COMPONENT CODE SUBCODE SUBCODE SUBCODE
0 9	C A 11 X 12 Z Z Z Z Z Z Z Z 16 Z 16 REVISION
	17 REPORT NUMBER 21 22 23 24 26 27 28 29 30 31 32
	ACTION FUTURE STREET SHUTDOWN HOURS 22 ATTACHMENT PORM SUB. PRIME COMP. COMPONENT MANUFACTURER SUBMITTED FORM SUB. SUPPLIER MANUFACTURER   X   (8)   X   (9)   C   (20)   Z   (21)   O   O   O   O   Y   (23)   N   (24)   N   (25)   Z   9   9   9   (26)
	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10	Evidence of scale formation at the interface indicates that the leakage
111	may have existed during prior periods of power operation. The leakage
1 2	stopped when the primary system pressure was reduced to atmospheric
1 3	conditions. Evaluation to ascertain corrective measures is underway.
1 4	90
7 8	STATUS OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32 N/A   C 31 During inspection following maintenance
	2 10 12 13 44 45 46 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36
1 6	Z 33 Z 34 NA
17	PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39  O O O 37 Z 38 NA
7 8	PERSONNEL INJUR 55 NUMBER DESCRIPTION 41
1 8	0 0 0 0 NA
	LOSS OF OR DAMAGE TO FACILITY 43
1 9	Z (2) NA 50
	PUBLICITY ISSUED DESCRIPTION 45
20	Y Jarjous responses to news media questions

Attachment to LER-79-018-01T-0 Consumers Power Co Big Rock Point Plant Docket 50-155

Following replacement of "o ring" seals on four control rod drive flanges, an inspection with primary system pressure at 900 psig and the reactor subcritical revealed water on the outside of the drive housing at F-2 at 0406 hours on 4-20-79. Since this could not positively be attributed to condensation, reactor vessel thermal insulation was removed and water was observed to be coming from the shrink fit interface between the reactor vessel and the F-2 drive housing with the primary system pressure at 500 psig at 1500 hours on 4-20-79. The leak rate average over a four hour period at about 900 psig was 8.7 milliliters per minute. When reactor pressure was reduced to atmospheric conditions, the leakage stopped.

The plant remains in shadown condition pending identification of the exact leak mechanism and subsequent evaluation. A project group has been organized to investigate the problem and the Nuclear Steam Supply System vendor and the reactor vessel fabricator have been contacted for assistance.

