

OPERATING DATA REPORT

DOCKET NO. 50-289  
 DATE May 11, 1981  
 COMPLETED BY D. G. Mitchell  
 TELEPHONE (717) 948-8553

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I  
 2. Reporting Period: April 1981  
 3. Licensed Thermal Power (MWt): 2535  
 4. Nameplate Rating (Gross MWe): 871  
 5. Design Electrical Rating (Net MWe): 819  
 6. Maximum Dependable Capacity (Gross MWe): 840  
 7. Maximum Dependable Capacity (Net MWe): 776  
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

\_\_\_\_\_

\_\_\_\_\_

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_

10. Reasons For Restrictions, If Any: \_\_\_\_\_

\_\_\_\_\_

\_\_\_\_\_

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>719.</u>	<u>2879.</u>	<u>58392.</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>0.0</u>	<u>31731.8</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>839.5</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>0.0</u>	<u>31180.9</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0.0</u>	<u>0.0</u>	<u>76531071.</u>
17. Gross Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>25484330.</u>
18. Net Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>23840053.</u>
19. Unit Service Factor	<u>0.0</u>	<u>0.0</u>	<u>53.4</u>
20. Unit Availability Factor	<u>0.0</u>	<u>0.0</u>	<u>53.4</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>0.0</u>	<u>51.9</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>0.0</u>	<u>49.9</u>
23. Unit Forced Outage Rate	<u>100.0</u>	<u>100.0</u>	<u>38.9</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

\_\_\_\_\_

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_\_

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289

UNIT TMI-I

DATE May 15, 1981

COMPLETED BY D. G. Mitchell

TELEPHONE (717) 948-8553

MONTH April, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April, 1981

DOCKET NO. 50-289  
 UNIT NAME TMI-I  
 DATE May 15, 1981  
 COMPLETED BY D. G. Mitchell  
 TELEPHONE (717) 948-8553

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	4/1/81	F	719	D	1				Regulatory Restraint Order

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

## OPERATIONS SUMMARY

The Unit was shutdown the entire month by order of the NRC. Core Cooling was provided by the Decay Heat System.

### MAJOR SAFETY RELATED MAINTENANCE

During the month, major safety related maintenance included:

#### Borated Water Storage Tank

Work consisted of adjusting the hanger on the eight (8) inch recirc line, completing the twenty-four (24) inch spool replacement, and filling the tank with borated water. Work on the twenty-four (24) inch spool piece progressed with the following work being performed.

1. Fitup of new spool and inspection by QC and Code Inspector.
2. Hot and Root weld pass.
3. Information x-ray of hot and root pass weld.
4. Weld repair of hot and root pass weld including information x-ray.
5. Weld out of welds.
6. Final x-ray.

The final x-ray results showed no indication in the weld.

Filling of the BWST started at the completion of the twenty-four (24) inch spool job with about forty-eight (48) feet of water pumped into the tank during the month.

Results of inspections on the eight (8) inch recirc flange and the twenty-four (24) inch spool showed no leaks were present. Filling of the BWST will progress into May.

#### Local Leak Rate Testing

Testing progressed this month with satisfactory results to date. Approximately 85 percent of the testing has been completed.

#### Emergency Feedwater Check Valves

Emergency Feedwater valves EF-V-12 A/B (check valves in EFW lines to OTSGs) were inspected. Minor repair work was required.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: \_\_\_\_\_

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).