

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE 4-15-81
 COMPLETED BY D. G. Mitchell
 TELEPHONE (717) 948-8553

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: March 1981
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.</u>	<u>2160.</u>	<u>57673.</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>0.0</u>	<u>31731.8</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>839.5</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>0.0</u>	<u>31180.9</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0.0</u>	<u>0.0</u>	<u>76531071.</u>
17. Gross Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>25484330.</u>
18. Net Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>23840053.</u>
19. Unit Service Factor	<u>0.0</u>	<u>0.0</u>	<u>54.1</u>
20. Unit Availability Factor	<u>0.0</u>	<u>0.0</u>	<u>54.1</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>0.0</u>	<u>52.6</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>0.0</u>	<u>50.5</u>
23. Unit Forced Outage Rate	<u>100.0</u>	<u>100.0</u>	<u>38.1</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-I
 DATE April 15, 1981
 COMPLETED BY D. G. Mitchell
 TELEPHONE (717) 948-8553

MONTH March 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

UNIT SHUTDOWNS AND POWER REDUCTIONS

50-289
 DOCKET NO. TH1-1
 UNIT NAME
 DATE April 15, 1981
 COMPLETED BY D. G. Mitchell
 TELEPHONE (717) 948-8553

REPORT MONTH March 1981

No.	Date	Type	Duration (Hours)	Reason	Method of Shutting Down Reactor	License Event Report #	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
1	3/1/81	F	744	D	1				Regulatory Restraint Order

- 1 F: Forced
 S: Scheduled
- 2 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)
- 3 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)
- 4 Exhibit G - Instructions for Preparation of Data Entry Sheets for License Event Report (LER) File (NUREG-0161)
- 5 Exhibit I - Same Source

OPERATIONS SUMMARY

The Unit was shutdown the entire month by order of the NRC. The Reactor Coolant System was cooled using the Decay Heat Removal System during the entire period.

MAJOR SAFETY RELATED MAINTENANCE

During the month the Unit remained in cold shutdown while restart activities progressed. The following major maintenance items were performed.

- A. Nuclear Services - River Water Pump overhaul (NR-P-1B)
- B. Snubber Tech Spec Surveillance
- C. Local Leak Rate Surveillance Testing
- D. Decay Heat Pump remote vent tie-in
- E. Borated Water Storage Tank work

Overhaul of the river water pump (NR-P-1B) included removal of motor, disassembly of pump, inspection of pump parts, reassembly of pump, installation of motor, motor to pump alignment, and testing of pump (IRD readings). Testing was performed with satisfactory results.

Snubber testing (Refueling Outage Surveillance) was completed during March. The snubber work included removal of snubbers, testing, repairing leaks, minor bleed rate adjustments and reinstallation of snubbers. Problems encountered were reported in Licensee Event Reports 81-02 and 81-04.

Local leak rate testing progressed through the month with one (1) Liquid Waste Disposal System Valve identified as requiring maintenance work. This valve will be repaired at a later date.

The restart modification on the Decay Heat Pumps remote vent system was installed, flush performed, and leak checked with satisfactory results.

Work on the Borated Water Storage Tank eight (8) inch recirc line flange and twenty-four (24) inch spool piece started with the following items being performed:

8 Inch Flange

- a. Bolts/nuts removed
- b. Flange seating surfaces inspected
- c. New gasket installed
- d. Bolts torqued

Following this work, minor leakage remained that is expected to be corrected by adjustment of the respective pipe hangers.

24 Inch Spool

- a. Existing spool piece removed
- b. Flanges removed from pipe
- c. Pipe prep'd for fitup of new spool

The remaining work will continue next month.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).