OYSTER CREEK **NUCLEAR GENERATING STATION**

JCP&L / GPU

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January 30, 1981

Mr. Boyce H. Grier, Director
Office of Inspection and Enforcement
Region I
United States Nuclear Regulatory Commission
641 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

SUBJECT: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report
Reportable Occurrence No. 50-219/81-01/3L

This letter forwards three copies of a Licensee Event Report to report Reportable Occurrence No. 50-219/81-01/3L in compliance with paragraph 6.9.2.b.1 of the Technical Specifications.

Very truly yours,

Ivan R. Finfrock, Jr.
Vice President - JCP&L
Director - Oyster Creek

IRF:dh
Enclosures

cc: Director (40 copies)
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Washington, D.C. 20555

Director (3)
Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D. C. 20555

NRC Resident Inspector (1)
Oyster Creek Nuclear Generating Station
Forked River, N. J.

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OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731

Licensee Event Report
Reportable Occurrence No. 50-219/81-01/3L

Report Date

January 30, 1981

Occurrence Date

January 2, 1981

Identification of Occurrence

During surveillance testing the Containment Spray high drywell pressure indicating switch IP-15C tripped at a value greater than that given in the Technical Specifications, Table 3.1.1, item E.1.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.b.1.

Conditions Prior to Occurrence

The plant was operating at steady state power.

Major Plant Parameters:

Power:	Reactor	1741 MWt
	Generator	602 MWe
Flow:	Recirculation	14.6×10^4 gpm
	Feedwater	6.42×10^6 lb/hr

Description of Occurrence

During performance of the "Containment Spray System Automatic Actuation Test" (surveillance procedure 607.3.002), the IP-15C trip point was found to exceed the Technical Specifications desired setpoint. Testing of the four sensors yielded the following data:

Switch Designation	Desired Setpoint	As Found (psig)	As Left (psig)
IP-15A	$\leq$2.0 psig	1.95	1.95
IP-15B	$\leq$2.0 psig	1.95	1.95
IP-15C	$\leq$2.0 psig	2.10	1.95
IP-15D	$\leq$2.0 psig	1.99	1.99

Apparent Cause of Occurrence

The cause of the occurrence was instrument repeatability. The switch was originally set at 1.94 psig and tripped at 2.10 psig. The range for repeatability is 2-3% of full range, which in the case of the IP-15 switches is 0.2-0.3 psig. The difference of 0.16 psig between the setpoint and the actual trip point clearly falls within the range of instrument repeatability.

Analysis of Occurrence

The Containment Spray System consists of two independent cooling loops, each of which is capable of removing heat from the primary containment in the event of a loss of coolant accident. The Containment Spray System will be initiated upon receipt of both a high drywell pressure signal and a reactor low-low water level signal.

Although switch IP-15C would have tripped at a slightly higher pressure than the desired setpoint, its actuation only would have been delayed by a fraction of a second. Also, the reactor low-low level setpoint is not reached until almost 4 seconds after the setpoint for high drywell pressure is reached. Due to this and the fact that switch IP-15A for the same instrument channel would have actuated at the required setpoint, the safety significance of the event is considered minimal.

Corrective Action

Pressure switch IP-15C was reset to trip within the Technical Specifications limit of 2.0 psig (as shown in the "As Left" values in the Description of Occurrence). The drift problem of these snap-action switches is being investigated, along with possible setpoint changes to account for instrument repeatability. The PORC has also recommended replacement of the switches with a more qualified model.

Failure Data

Manufacturer: ITT Barton
Model: #288A pressure indicating switch
Range: 0-10 psig