

Geriecal Offices: 212 West Michigan Avenue, Jackson, Nichigan 49201 + Area Code 517 788-0550

August 31, 1977

Regulatory Esc. CV

Director of Nuclear Reactor Regulation Att: Mr Don K Davis, Acting Branch Chief Operating Reactor Branch No 2 US Nuclear Regulatory Commission Washington, DC 20555

pdi

DOCKET 50-155 - LICENSE DPR-6 -BIG ROCK POINT PLANT -CHANGE IN MCHFR LIMITS

Enclosed is an Exxon Nuclear Corporation study which provides a synthesized Hench-Levy Minimum Critical Heat Flux Ratio (MCHFR) limit which ensures that a Minimum Critical Power Ratio (MCFR) limit of 1.32, as derived by the XN-2 Critical Power Correlation, will not be violated at Big Rock Point. This study is forwarded in response to your letter of August 3, 1977.

David A Bixel Nuclear Licensing Administrator

CC: JGKeppler, USNRC

8102040677

772500648

HENCH-LEVY MCHFR LIMIT FOR THE BIG ROCK POINT REACTOR

SUMMARY

A minimum critical heat flux ratio (MCHFR) limit of 2.15 as determined by the ENC synthesized Hench-Levy critice? heat flux correlation has been shown to be conservative in the assessment of thermal margins for the Big Rock Point Reactor. The 2.15 MCHFR limit was determined to conservatively provide nonpenetration of the 1.32 minimum critical power ratio (MCPR) as determined by the XN-2 critical power correlation.⁽¹⁾ The analysis, performed with the XCOBRA computer code,⁽²⁾ considered the effects of variations in power/flow, assembly local peaking, axial power distribution and nominal expected conditions. The 2.15 MCHFR limit was determined to be applicable for the limiting assembly and envelopes the anticipated operating range of axial power distribution, inlet enthalpy, local peaking, and mass velocity for Crele 15 and future cycles.

The 2.15 MCHFR limit we shown to be applicable for conditions of reduced reactor flow as long as the nominal power-to-flow relationship is maintained; that is the MCPR was determined to be greater for the partial power/partial flow cases than the full power/full flow case. The analysis assumes that the reactor is operated such that a reduction of reactor flow is accompanied by a proportionate reduction in reactor power.

THERMAL HYDRAULIC ANALYSIS

- The analysis presented here is necessitated by the desire to relate the METER as determined by the ENC synthesized Hench-Levy critical heat flux correlation with the MCPR as determined by the NRC approved XN-2 critical power correlation. The analysis consists of several sensitivity studies which establish an MCHFR limit that conservatively provides nonpenetration of the 1.32 XN-2 MCPR limit.

A power-flow sensitivity was performed to determine the most limiting conditions for Cycle 15 and subsequent cycles in terms of full power/full flow or partial power/partial flow at the reactor operating design conditions shown in Table 1. As shown in Table 2, the most limiting condition was determined to be full power/full flow. The additional sensitivity studies were then performed at conditions equivalent to those given in Table 1 to establish the required Hench-Levy MCHFR limit value which envelopes anticipated operating conditions for Cycle 15 and subsequent cycles.

An assembly local peaking sensitivity study was performed assuming the most adverse anticipated conditions of assembly local peaking and assembly axial power distribution as well as ± 50% variation in inlet subcooling enthalpy. For each value of inlet enthalpy and mass velocity, those fuel assembly powers which resulted in an XN-2 MCPR of 1.32 were determined. The Hench-Levy MCHFR for each case was then calculated using the conditions obtained above. As is shown in Table 3, the maximum Hench-Levy MCHFR so obtained was 2.11. Thus, for additional conservatism a Hench-Levy MCHFR limit value of 2.15 is proposed.

The sensitivity of the Hench-Levy MCHFR limit to axial power distribution was determined in this study. This sensitivity is shown by comparison of the local peaking sensitivity results (Table 3) with the results of a similar set of calculations which assumed an axial peaking of 1.5 at X/L = 0.5. All other is sembly conditions were identical to those used in the local peaking sensitivity study. The determination of the MCPR limiting assembly power and corresponding Hench-Lesy MCHFR values was identical to the procedure used in the local peaking study. As shown in Table 4, the highest Hench-Levy so obtained was 2.06 and further illustrates that the proposed value of 2.15 conservatively envelopes anticipated Cycle 15 and subsequent cycle conditions.

The nominal expected conditions study was performed to assess the conservatism of a 2.15 ENC synthesized Hench-Levy MCHFR limit at the conditions indicated in Table 1. An ENC synthesized Hench-Levy MCHFR limit of 1.60 was found to be adequate, further demonstrating the acceptability of the proposed 2.15 limit. The results of this sensitivity study are shown in Table 5.

ASSUMED OPERATING CONDITIONS

Reagtor Power @ 122% Overpower, MWth	292.8
Effective Reactor Flow Rate, 10 ⁶ lb/hr	9.9
Inlet Subcooling, Btu/lb	22.8
Operating Pressure, psia	1350
Hot Assembly Operating Conditions*	
Hot Assembly Flow Rate, 1b/hr	123,100
Radial Feaking	1.45
Local Peaking	1.20
Assembly Power @ 122% Overpower, MW	5.05

F", "

SENSITIVITY OF THERMAL MARGINS TO PARTIAL POWER/FLOW OPERATION

· ----

×.

		Hot Assembly Thermal Margin	
Reactor Power (MW _{th})	Effective Reactor Flow (1b/hr)	ENC-Hench-Levy MCHFR	XN-2 MCPR
292.8	9.9 x 10 ⁶	1.86	1.52
2140	9.9 x 10 ⁶	2.63	1.87
180	7.4 x 10 ⁶	3.55	2.25
120	5.0 x 10 ⁶	5.40	2,80
	Reactor Power (MW _{th}) 292.8 240 180 120	Reactor Power (MW_{th}) Effective Reactor Flow (1b/hr)292.8 9.9×10^6 240 9.9×10^6 180 7.4×10^6 120 5.0×10^6	Reactor Power (MW_{th}) Effective Reactor Flow (1b/hr)Hot Assembly The ENC-Hench-Levy MCHFR292.8 9.9×10^6 1.86 240 9.9×10^6 2.63 180 7.4×10^6 3.55 120 5.0×10^6 5.40

·

SENSITIVITY OF THERMAL MARGINS TO ASSEMBLY LOCAL POWER CONDITIONS

Assembly Mass Velocity (1b/hr- ft ²)	Assembly Flow (1b/hr)	Inlet Enthalpy (Btu/lb)	Assembly Power (MW)	Hench-Levy MCHFR	XN-2 MCPR
0.79 x 10 ⁶	127,900	560	3.66	2.00	1.32
0.91 x 10 ⁶	148,600	570	3.57	2.03	1.32
1.12 x 10 ⁶	182,800	580	3.40	2.11	1.32

1.70 axial peak @ 0.7 of core height.

1.60 local peaking factor.

SENSITIVITY OF THERMAL MARGINS TO ASSEMBLY AXIAL PEAKING CONDITIONS

.

1.1

Assem'__y Mass Hench-Levy XN-2 Assembly Power Inlet Enthalpy Velocity (1b/hr-Assembly Flow (MW) MCHFR MCPR ft2) (1b/hr) (Btu/1b) 0.79 x 10⁶ 1.94 1.32 4.22 560 127,900 0.91 x 10⁶ 1.97 1.32 4.10 148,600 570 1.12 x 10⁶ 2.06 1.32 3.88 182,800 580

1.51 axial peaking @ 0.5 of core height.

1.60 local peaking factor.

TABLE >

SENSITIVITY	OF	THERM	IAL	MARC	JINS	AT
NOMINAL	CSE	MBLY	CON	DITI	IONS	

Assembly Mass Velocity (1b/hr-ft ²)	Feak Assembly Flow (lb/hr)	Inlet Enthalpy (Btu/1b)	Hench-Levy MCHFR*	XN-2 MCPR*
0.65 x 10 ⁶	106,200	560	1.60	1.45
0.70 x 10 ⁶	114,400	570	1.60	1.47
0.77 x 10 ⁶	12%,700	580	1.60	1.48

*Corresponding to reactor operation at 122% overpower as shown in Table 1. 1.51 axisl peaking at .5 of core height. 1.20 local peaking factor.

REFERENCES

.

.

1. XN-75-34, "The XN-2 Critical Power Correlation," Revision 1 (1975).

2. XN-NF-77-43, "XCOBRA Code Operating Manual" (1977).

C FORM 195	FORM 195 U.S. NUCLEAR REGULATORY COMMISSION			DOCKET NUMBER	
Mr. Don K. Davis		FROM: Cons Jacl Dav	sumers Power Co. kson, Michigan 49201 id A. Bixel	DATE OF DOCUMENT 08/31/77 DATE RECEIVED 09/06/77	
DRIGINAL	NOTORIZED KUNCLASSIFIED	PROP	INPUT FORM	I Sig NeD	
CRIPTION			ENCLOSURE Consists of Ex providing synthesized He Heat Flux Ratio limit.	xxon Nuc. Corp. study ench-Levy Minimum Critical	
		1p	9p		
ACK	NOWLEDGED				
	DO NOT	REMOVE			
ANT NAME: BI	G ROCK POINT				
jcm 09/06	/77		L an End	Parti	
SAFETY		FOR ACTIC	ON/INFORMATION	PLC D	
BRANCH CHIEF:). <u>s</u>			
		INTERNA	L DISTRIBUTION		
REG FILE					
TEF(2)					
OFLD					
HANAUER					
CHECK					
STELLO					
EISENHUT					
SHAO					
BAER					
BUTLER _					
GRIMES					
J. COLLINS .					
1 2					
	EXTERN		ION	CONTROL NUMBER	
LPDR: CLAS	te unt Lie	h.		1	
TIC	1 733			11.4. 1.	
NSIC				172500048	
16 CYS ACRS	SENT CATEGORY	A 40			
The news					
the state of the second s		Statement Statement and Statement and Statement	and the second		