

UNITED STATES NUCLEAR REGULATORY COMMISSION REGION III 2443 WARRENVILLE ROAD, SUITE 210 LISLE, ILLINOIS 60532-4352

December 11, 2019

Mr. Dean Curtland Director of Site Operations NextEra Energy Duane Arnold, LLC 3277 DAEC Road Palo, IA 52324-9785

SUBJECT: DUANE ARNOLD ENERGY CENTER – TRIENNIAL INSPECTION OF EVALUATION OF CHANGES, TESTS AND EXPERIMENTS BASELINE INSPECTION REPORT 05000331/2019013

Dear Mr. Curtland:

On December 5, 2019, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Duane Arnold Energy Center and discussed the results of this inspection with you and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <u>http://www.nrc.gov/reading-rm/adams.html</u> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/**RA**/

Robert C. Daley, Chief Engineering Branch 3 Division of Reactor Safety

Docket No. 05000331 License No. DPR-49

Enclosure: As stated

cc w/ encl: Distribution via LISTSERV®

Letter to Mr. Dean Curtland from Robert C. Daley darted December 11, 2019.

SUBJECT: DUANE ARNOLD ENERGY CENTER – TRIENNIAL INSPECTION OF EVALUATION OF CHANGES, TESTS AND EXPERIMENTS BASELINE INSPECTION REPORT 05000331/2019013

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U.S. NUCLEAR REGULATORY COMMISSION Inspection Report

| Docket Number: | 05000331 |
|------------------------|--|
| License Number: | DPR-49 |
| Report Number: | 05000331/2019013 |
| Enterprise Identifier: | I-2019-013-0018 |
| Licensee: | NextEra Energy Duane Arnold, LLC |
| Facility: | Duane Arnold Energy Center |
| Location: | Palo, Iowa |
| Inspection Dates: | December 2, 2019 to December 5, 2019 |
| Inspectors: | G. Hausman, Senior Reactor Inspector J. Robbins, Reactor Inspector N. Shah, Project Engineer |
| Approved By: | Robert C. Daley, Chief Engineering Branch 3 Division of Reactor Safety |

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a triennial inspection of evaluation of changes, tests and experiments baseline inspection at Duane Arnold Energy Center, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to https://www.nrc.gov/reactors/operating/oversight.html for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at

<u>http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html</u>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.17T - Evaluations of Changes, Tests, and Experiments

Sample Selection (IP Section 02.01) (28 Samples)

The inspectors reviewed the following evaluations, screenings, and/or applicability determinations for 10 CFR 50.59 from 2015 to 2019.

- (1) AR 02175375-01, Power System Stabilizer Removal from Service, January 16, 2017
- (2) CA 02102327-01, Placement of Portable Heaters in the Motor Generator Set Room, July 1, 2016
- (3) EC 0000281703, Chromalox Replacement Project, Revision 0
- (4) EC 0000282458, River Water System Cable Replacement, Revision 0
- (5) EC 0000287931, Implementation of Revised Calculations for Loss of Coolant Accident Dose Consequences, Revision 0
- (6) EC 0000287997, Residual Heat Removal Discharge Header Pressure Switch Setpoint, Revision 1
- (7) EC 0000156026, Replace Existing Reactor Feedwater Pumps (RFP) and Motors with RFP and Motors of Larger Capacity, Revision 43
- (8) EC 0000283115, Flow Indication Controller (FIC2309) Replacement Siemens 353 Controller from Design Level A to B, Revision 1
- (9) EC 0000283472, Fukushima Spent Fuel Pool Level Instrumentation, Revision 6
- (10) EC 0000283785, Replace Obsolete Standby Gas Treatment System Flow Indication Controllers (FIC5828A/B), Revision 2
- (11) EC 0000283960, Raise All Level Setpoints for Limit Switch LS3701 to Allow Pump Starts Due to Sticking Level Element Float LE3701 at Lowest Level, Revision 0
- (12) EC 0000285459, Connect Diesel Air Compressor to Plant Air System, Revision 0
- (13) EC 0000285499, Calculation CAL-E95-015 Calibration Frequency Revised, Revision 1
- (14) EC 0000285928, Programmable Logic Controller (PLC1701C) Replacement, Revision 1
- (15) EC 0000286670, Compressor Motor (1K001-M) Tripped, Overload Jumper-ed Out (Bypassed), Revision 1
- (16) EC 0000287838, Revise Temperature Element (TE4403) Setpoint on Temperature Recorder (TR4400) to Provide Annunciation with Temperature Rise, Revision 0
- (17) EC 0000287965, Disable Low Oil Level Alarm XR2 to Prevent Masking Other Alarms, Revision 0

- (18) EC 0000288156, Replace Obsolete Control Building Chiller Temperature Controllers (TC6924A/B), Revision 2
- (19) EC 0000288815, Diesel Air Compressor Back-Up to Plant Air System, Revision 0
- (20) EC 0000289081, Revise CAL-M91-014 To Be in Compliance with ANSI 195-1976, Section 5.4, Revision 1
- (21) EC 0000291031, Reconfigure Limit Switch (LS2219) Logic to Remove Automatic Opening of Solenoid Valve (SV2219), Revision 2
- (22) EC 0000291104, Replace Level Element (LE-3768) with Longer Length Level Element, Revision 3
- (23) Procedure Change Request (PCR 02073808), Annunciator Response Procedure ARP-1C22 Hydrogen/Oxygen Control Panel, Revision 40
- (24) Procedure Change Request (PCR 02102503), Abnormal Operating Procedure AOP-903 Severe Weather, Revision 55
- (25) Procedure Change Request (PCR 02266292), Operating Instruction OI 878.1 - Source Range Neutron Monitoring System, Revision 21
- (26) Procedure Change Request (PCR 02268679), Annunciator Response Procedure ARP-1C22 Hydrogen/Oxygen Control Panel, Revision 42
- (27) UFSAR Change Request (UCR#: 2016-007), Revise Discussion Regarding Location of Steam Leak Detection Temperature Elements, Revision 24
- (28) UFSAR Change Request (UCR#: 2019-007), Revise Main Steam Line Isolation Valve Position Indication Description, August 22, 2019

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors confirmed that proprietary information was controlled to protect from public disclosure.

• On December 5, 2019, the inspectors presented the triennial inspection of evaluation of changes, tests and experiments baseline inspection results to Dean Curtland and other members of the licensee staff.

DOCUMENTS REVIEWED

| Inspection Procedure | Туре | Designation | Description or Title | Revision or Date |
|-------------------------|--------------------------------|----------------|---|---------------------|
| 71111.17T | Calculations | CAL-E98-004 | Setpoint Calculation High Reactor Vessel Level, HPCI, RCIC, Feedwater, and Turbine Trip in Remote Shutdown – LS4540 | 2 |
| | | CAL-MC-138 | Diesel Oil Storage Tank Level Switch | 3 |
| | | CAL-R17-002 | Dose Rate Evaluation of Reactor Vessel Water Levels During Refueling for EAL Thresholds | 0 |
| | Corrective Action Documents | AR 01988025 | NRC CGD [Commercial Grade Dedication] Inspection Issue of Third Party Dedication | 06/22/2015 |
| | | AR 02100600 | | |
| | | AR 02102327 | 2016 50.59: Eval of Portable Heaters During Severe Weather | 08/08/2016 |
| | | AR 02102503 | AOP 903 - Severe Weather | 01/14/2016 |
| | | AR 02102754 | 2016 50.59: Water Temperature Input to Mark I Analysis | 02/22/2016 |
| | | AR 02103084 | 2016 50.59: Inspection Identified Inadequate 50.59 Screening | 01/26/2016 |
| | | AR 02115464 | RCIC Room Steam Leak Detection Not Consistent with CLB [Current Licensing Basis] | 10/02/2016 |
| | | AR 02138974 | T.S. Bases Change Request: B3.3.6.1 | 12/14/2016 |
| | | AR 02138981 | UFSAR Change Request: Chapter 7 | 10/12/2016 |
| | | AR 02139054 | Evaluate Process Utilized to Move STP 3.5.1-03A/B to Online | 09/23/2016 |
| | | AR 02175375 | Power System Stabilizer 50.59 | 01/18/2017 |
| | | AR 02332637 | 2019 NRC 50.59/Mod Pre-Inspection Identified Items | 10/24/2019 |
| | | AR-LIC 2138981 | DAEC UFSAR Chapter 7 | 06/20/2016 |
| | | PCR 02073808 | Annunciator Response Procedure ARP-1C22 Hydrogen/Oxygen Control Panel | 40 |
| | | PCR 02090816 | Bases-ATWS (Anticipated Transient Without Scram) | 19 |
| | | PCR 02102503 | AOP 903 Severe Weather, Revision 55 | 01/14/2016 |
| | | PCR 02225747 | | |
| | | PCR 02259266 | Biennial Review – ATWS - RPV Control, | 0 |
| | | PCR 02266292 | OI 878.1 - Source Range Neutron Monitoring System | 21 |

| Inspection Procedure | Туре | Designation | Description or Title | Revision or Date |
|-------------------------|--|----------------|---|------------------|
| | | PCR 02268679 | Annunciator Response Procedure ARP 1C22 - Hydrogen/Oxygen Control Panel | 42 |
| | | PCR 02307806 | AOP 302.1 - Loss of 125 Vdc Power | 60 |
| | Corrective Action Documents Resulting from Inspection | AR 02337533 | 50.59 Screening for EC 290224 | 12/05/2019 |
| | Engineering Changes | CA 02102327-01 | Placement of Portable Heaters in the MG [Motor Generator] Set Room | 07/01/2016 |
| | | EC 0000156026 | Replace Existing Reactor Feedwater Pumps (RFP) and Motors with RFP and Motors of Larger Capacity | 43 |
| | | EC 0000283115 | FIC2309 Replacement Siemens 353 Controller from Design Level A to B | 1 |
| | | EC 0000283472 | Fukushima Spent Fuel Pool Level Instrumentation | 6 |
| | | EC 0000283785 | SBGT 1V-SGT-1A\B Flow Ind Controller Replacement Siemens 353 Controller from Design Level A to B FIC5828A/B | 2 |
| | | EC 0000283960 | Raise All Level Setpoint for LS3701 to Allow Pump Starts Due to Sticking LE3701 Float at Lowest Level | 0 |
| | | EC 0000285459 | Connect Diesel Air Compressor to Plant Air System | 0 |
| | | EC 0000285499 | CAL-E95-015 Calibration Frequency Revision | 1 |
| | | EC 0000285928 | PLC 1701C Replacement | 1 |
| | | EC 0000286670 | 1K001-M Compressor Motor Overload Trip Jumper-ed Out (Bypassed) | 1 |
| | | EC 0000287838 | Revise TE4403 Setpoint on TR4400 to Provide Annunciation with Temperature Rise | 0 |
| | | EC 0000287965 | Disable XR2 Low Oil Level Alarm to Prevent Masking Other Alarms | 0 |
| | | EC 0000287997 | RHR [Residual Heat Removal] Discharge Header Pressure Switch Setpoint | 1 |
| | | EC 0000288156 | TC6924A/B Control Building Chiller Temperature Controller Siemens 353 Controller from Design Level A to B | 2 |
| | | EC 0000288815 | Diesel Air Compressor Back-Up to Plant Air System | 0 |
| | | EC 0000289081 | Revise CAL-M91-014 To Be in Compliance with | 1 |

| Inspection Procedure | Туре | Designation | Description or Title | Revision or Date |
|-------------------------|----------------------------|-----------------|--|-------------------------------------|
| | | | ANSI 195-1976, Section 5.4 | Duto |
| | | EC 0000291031 | Reconfigure LS2219 Logic to Remove Automatic Opening of SV2219 | 2 |
| | | EC 0000291104 | Replace LE-3768 with Longer Length Level Element | 3 |
| E | Engineering Evaluations | ACE 01949946 | 2016 50.59: Eval of Portable Heaters During Severe Weather | 02/16/2016 |
| | | EC 0000281703 | Chromalox Replacement Project | 0 |
| | | EC 0000282458 | RWS Cable Replacement | 0 |
| | | EC 0000287931 | Implementation of Revised Calculations for LOCA Dose Consequences | 0 |
| | Miscellaneous | IP 71111.17T | Evaluations of Changes, Tests, and Experiments | 01/01/2017 |
| | | NEI 96-07 | Guidelines for 10 CFR 50.59 Implementation | 1 |
| | | UCR #: 2016-007 | Revise Discussion Regarding Location of Steam Leak Detection Temperature Elements | 24 |
| | | UCR #: 2019-007 | Revise MSIV [Main Steam Line Isolation Valve] Position Indication Description | 08/22/2019 |
| | Procedures | EN-AA-203-1100 | Engineering Evaluations | 13 |
| | | EN-AA-203-1201 | 10 CFR Applicability and 5059 Screen Reviews | 17 |
| | | EN-AA-203-1202 | 10 CFR 5059 Evaluation | 3 |
| | | EN-AA-205-1102 | Temporary Configuration Changes | 15 |
| | Self-Assessments | AR 02276319 | 50.59/Modification Pre-Inspection Assessment | 12/02/2019 through 12/06/2019 |
| | | PDA 19-001 | Duane Arnold Nuclear Assessment Report - Engineering | 03/28/2019 |