OPERATIONS SUMMARY

November 1980

At the beginning of the reporting period, the Oyster Creek Nuclear Generating Station was operating at 96.3% capacity (628 Mwe). Load was limited by condensate demineralizer differential pressure. The ultimate cause of the differential pressure limitation was the inability of the New Radwaste System to process liquid and solid waste as designed.

Load was reduced temporarily on November 4, 1980 for an ABRO on a condensate demineralizer. On November 7, 1980 load was again reduced for maintenance on "C" feedwater string and #4 turbine stop valve. After maintenance, full load was reached on November 10, 1980. The plant was shut down on November 21, 1980 for maintenance on "C" High Pressure Feedwater Heater and the North Feedwater Line Check Valve in the Trunion Room. The reactor was started up on November 28, 1980 but delayed through to November 29, 1980 due to problems with containment inerting. The generator came on line on November 29, 1980 and load was being increased at the end of the reporting period.

There were 9 Reportable Occurrences and 3 Non-Routine Environmental Operating Reports during the month:

RO# 80-47 occurred on November 1, 1980 when Core Spray High Drywell Pressure Switches R.V. 46 A, B, C, and D were found to have settings less conservative than Technical Specification requirements.

RO# 80-48 occurred on October 29, 1980 when snubber 23-7 failed functional testing.

RO# 80-49 occurred on November 5, 1980 when Isolation Condenser "B" vent valves were found to be inoperable and Isolation Condenser was inoperable.

RO# 80-50 occurred on November 6, 1980 when Containment Spray High Drywell Pressure Switches IP 15 A, B, C, and D were found to have settings less conservative than Technical Specification requirements.

RO# 80-51 occurred on November 8, 1980 when the daily surveillance for APLHGR, LLHGR, and MCPR was not performed.

RO# 80-52 occurred on November 18, 1980 when Reactor Triple Low Water Level Indicator Switches RE 18A and D were found to have settings less conservative than Technical Specification requirements.

RO# 80-53 occurred on November 30, 1980 when CRD Pumps A and B were made inoperable at separate times to repair a seal water leak.

RO# 80-54 occurred on November 30, 1980 when Core Spray Pumps A and C were temporarily inoperable at separate times when wet by leakage from the CRD pumps.

NREOR# 80-10 occurred on November 11, 1980 when less than two dilution pumps were operating for a period of 25 minutes with ambient intake water temperature less than 60.0°F.

NREOR# 80-11 occurred on November 18, 1980 when ambient intake water temperature dropped resulting in a corresponding drop in the discharge temperature which caused a typical fish kill.

NREOR# 80-12 occurred on November 11, 1980 when the plant shut down and the resulting discharge temperature drop killed a number of fish in the surrounding waters.

OPERATING DATA REPORT

OPERATING STATUS

UNIT NAME...OYSTER CREEK

DOCKET NUMBER...50-219

UTILITY DATA PREPARED BY...J.B. SKLAR 609-693-6013

REPORTING PERIOD... November 1980

LICENSED THERMAL POWER(MWT)...1930

NAMEPLATE RATING(GROSS MWE)...650

DESIGN ELECTRICAL RATING(NET MWE)...650

MAXIMUM DEPENDABLE CAPACITY(GROSS MWE)...650

MAXIMUN DEPENDABLE CAPACITY(NET MWE)...620

IF CHANGES OCCUR IN CAPACITY RATING SINCE LAST REPORT, GIVE REASON...
NONE

POWER LEVEL TO WHICH RESTRICTED, IF ANY(NET MWE)... NO RESTRICTION

REASON FOR RESTRICTION, IF ANY...
NO RESTRICTION

	MONTH	YEAR	CUMULATIVE
HOURS IN PERIOD	720.0	8039.0	95903.0
HOURS RX CRITICAL	563.9	3046.3	71685.6
RX RESERVE SHUTDOWN HRS.	0.0	0.0	468.2
HRS. GEN ON LINE	535.3	2918.5	70223.9
UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
GROSS THERMAL ENERGY	926688.0	4942321.2	118821480.5
GROSS ELEC ENERGY	309270.0	1593960.0	40476275.0
NET ELEC ENERGY	296410.0	1522558.0	39003571.0
UT SERVICE FACTOR	74.3	36.3	73.2
UT AVAILABILITY FACTOR	74.3	36.3	73.2
UT CAPACITY FACTOR MDC	66.4	30.5	67.1
UT CAPACITY FACTOR DER	63.3	29.1	62.6
FORCED OUTAGE FACTOR	25.7	12.2	6.7

THE NEXT SCHEDULED OUTAGE IS TO BEGIN ON SPRING OF 1981

AVERAGE DAILY POWER LEVEL

DOCKET #..... 50-219
UNIT..... O. C. #1
REPORT DATE... December 10, 1980
COMPILED BY... J.B. SKLAR
TELEPHONE..... 609-693-6013

MONTH November 1980

DAY	MW	DAY	MW
1.	602.	17.	604.
2.	604.	18.	606.
3.	604.	19.	603.
4.	582.	20.	600.
5.	513.	21.	583.
6.	588.	22.	15.
7.	593.	23.	0.
8.	335.	24.	0.
9.	453.	25.	0.
10.	579.	26.	0.
11.	602.	27.	0.
12.	602.	28.	0.
13.	603.	29.	9.
14.	602.	30.	302.
15.	604.		
16.	604.		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-219
UNIT NAME O.C. #1
DATE December 9, 1980
COMPLETED BY J. B. Sklar
TELEPHONE 609-693-6013

REPORT MONTH November, 1980

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	11-7-80	F	0.0	B	5	N/A	ZZ	ZZZZZ	Leaking drain valves on "C" feedwater pump and a flow control valve were replaced. Maintenance was also performed on #4 Turbine stop valve and 1B2 moisture removal valve.
7	11-21-80	F	184.75	B	1	N/A	ZZ	ZZZZZ	Plugged 27 leaking tubes in 1C3 HP Feedwater Heater and repaired seal weld on feed check valve check valve hinge pin.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

MONTHLY QASL MECHANICAL MAINTENANCE REPORT

System	Malfunction	Corrective Action
Emergency Diesel Generator #1	Cooling air fan securing bolt failed from fatigue.	Replaced bolt.
A CRD Pump	Outboard Bearing oil leak	Repaired Oiler.
A CRD Filter	High ΔP across filter	Replaced Filter elements
CRD42-11	V-106 valve leaks past seat	Installed new ball and packing.
CRD42-23	V-106 valve leaks past seat	Installed new ball and packing.
CRD 42-07	V-106 valve leaks past seat	Installed new ball and packing.
Emergency Condenser system	V-14-1, 5, 19, 20, 7, 3, 4, 8 packing leaks	Repacked all valves.
CRD 38-19	Packing leak on V-107	Adjusted packing.
CRD 02-27	V-127 leaks past seat	Replaced disc.
CRD Hydraulic Control	V-15-26 and 25 packing leaks	Adjusted packing.
Reactor Recirculation Sample Line	V-24-29 diaphragm leaks	Replaced diaphragm
Emergency Condenser	V-14-30 packing leaks	Repacked valve
CRD 34-31	V-127 leaking past seat	Replaced seat and gaskets
"B" CRD Filter	High ΔP	Replaced filters.
CRD 42-07	V-111 malfunction	Replaced valve.

NOVEMBER SUMMARY OF QASL ELECTRICAL MAINTENANCE

Equipment	Malfunction	Corrective Action
Isolation Condenser Vent Valves	Failed to operate	Replaced valves, troubleshoot entire circuit and verified operability.
Core Spray Booster Pump Motors	Wiring entrances not water-proofed adequately	New gaskets for covers installed and waterproofing foam used to seal conduits.
Condensate Transfer Pump 1 - 2	Contactor Chattering	Cleaned contactor and checked current through power leads.
Emergency Service Water Pump Motors	Corroded Filters	Replaced with stainless steel type filters.
Fire Protection Alarm	Defective ZH-30 Module	Replaced.
Diesel Generator 1 - 2	Fuel transfer pump motor	Replaced motor starting switch (in motor)
Diesel Generator 1 - 2	Voltage regulator rectifier stack assembly (CR5) faulty	Replaced rectifier.
Diesel Generator 1 - 2 Battery Charger	Relay STYA faulty	Replaced.
Diesel Generator 1 - 2	Battery Charger	Cleaned burned contact on STYA relay.
Diesel Generator 1 - 2	Battery Charger	Installed temporary charger.
Diesel Generator 1 - 1	Mail breaker light indication out	Replaced light socket (open indication)
Service Water Pump 1 - 1	Motor required replacement	Replaced.
Isolation Condenser Valve V-14-30	Double Indication	Adjusted valve timing.
Main Steam Valves V-1-110	Valve indicates full shut constantly	Torque switch replaced on V-1-110
CRD Pump Motor	Trips out on overload	Replaced overload in breaker.

NOVEMBER SUMMARY OF QASL INSTRUMENT MAINTENANCE

Equipment	Malfunction	Corrective Action
Drywell Ventillation Isolation Bypass	Received Alarm	Cleaned contacts of 6K60 and 6K61
Fuel Pool Low Level Alarm	Alarm Indications	Adjusted Bubbler flow.
SRM Channel 23	Period Indication failed	Replaced control module.
IRM Channel 14	Recorder indication different than meter.	Adjusted recorder
APRM Channel 3	Pegged downscale	Replaced a diode in trip aux. unit
Reactor Vessel Temp. Recorder	All points printed upscale	Stepping solenoid bearings cleaned and oiled.
Area Radiation Monitor - Containment Spray Heat Exchanger D	Failed upscale	Replaced faulty detector
Rod Position Indicator 14-11	Missing pos. 42, 43, 46, and 47 due to open lead	Replaced problem and installed jumper
Isol Cond "B" press indicator	Out of calibration	Replaced and calibrated indicator
"A" Recirc Flow Indicator	Out of calibration	Replaced and calibrated indicator
Accoustic Safety & Relief Valve Indicator	No audible response from NR-108A and NR-28N	Tightened cable connector and replaced accelerometer respectively.
ERV-A	ERV-A opened @ approximately 800 psi	Found pressure switch would trip above and below 1000# (i.e., 1050# , and 865#) - Replaced pressure switch assembly.

Oyster Creek Station #1
Docket No. 50-219

REFUELING INFORMATION -

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: November 28, 1981

Scheduled date for restart following refueling: May 31, 1982

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

No Technical Specification change relative to the refueling is anticipated.

Scheduled date(s) for submitting proposed licensing action and supporting information:

No submittals are scheduled.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1) General Electric fuel assemblies - fuel design & performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2) Exxon Fuel Assemblies - No major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core - 560
(b) in the spent fuel storage pool - 781

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800 Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The Spring 1987 Outage