



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
631 PARK AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

November 10, 1980

Docket No. 50-485

Rochester Gas and Electric Corporation
ATTN: Mr. J. E. Arthur
Chief Engineer
89 East Avenue
Rochester, New York 14649

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REGISTRATION

Gentlemen:

The enclosed IE Information Notice No. 80-41, "Failure of Swing Check Valve in the Decay Heat Removal System at Davis-Besse Unit No. 1," is forwarded to you for information. No written response is required. If you desire additional information regarding this matter, please contact this office.

Sincerely,

Boyce H. Grier
Director

Enclosures:

- 1. IE Information Notice No. 80-41
- 2. List of Recently Issued IE Information Notices

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cc w/encls:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

SSINS No.: 6835
Accession No.:
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IN 80-41

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November 10, 1980

IE Information Notice No. 80-41: FAILURE OF SWING CHECK VALVE IN THE DECAY
HEAT REMOVAL SYSTEM AT DAVIS-BESSE UNIT
NO. 1

Description of Circumstances:

On October 9, 1980 the resident inspector at the Davis-Besse facility was informed that the licensee had performed leak rate tests and identified excessive leakage through Decay Heat Removal System check valve CF-30. Valve CF-30 is the inboard one of two in-series check valves that is used to isolate the reactor coolant system from the low pressure decay heat removal system. On further investigation the licensee found that the valve disc and arm had separated from the valve body and was lodged just under the valve cover plate. The two 2 5/8 x 5/8 inch bolts and locking mechanism for the bolts that holds the arm to the valve body were missing and have not been located. The CF-30 valve is a 14-inch swing check valve manufactured by Velan Valve Corporation. The cause of the failure has not been identified.

The test that was in progress when the failure was identified was being conducted in response to a letter from Darrell G. Eisenhut, Office of Nuclear Reactor Regulations to all Light Water Reactor Licensees, LWR Primary Coolant System Pressure Isolation Valves, February 23, 1980.

This IE Information Notice is provided as an early notification of a possibly significant matter that is still under review by the NRC staff. It is expected that recipients will review the information for possible applicability to their facilities. No specific action or response is requested at this time. If NRC evaluations so indicate, further licensee actions may be requested or required.

No written response to this IE Information Notice is required. If you have any questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

RECENTLY ISSUED
IE INFORMATION NOTICES

Information Notice No.	Subject	Date Issued	Issued to
80-40	Excessive Nitrogen Supply Pressure Activates Safety-Relief Valve Operation to Cause Reactor Depressurization	11/6/80	All holders of a power reactor OL or CP
80-38	Cracking in Charging Pump Casing Cladding	10/30/80	All holders of a PWR power reactor OL or CP
80-37	Containment Cooler Leaks and Reactor Cavity Flooding at Indian Point Unit 2	10/24/80	All holders of a power reactor OL or CP
80-36	Failure of Steam Generator Support Bolting	10/10/80	All holders of a power reactor OL or CP
80-35	Leaking and Dislodged Iodine-125 Implant Seeds	10/10/80	All holders of a Category G or G1 Medical License
80-34	Boron Dilution of Reactor Coolant During Steam Generator Decontamination	9/26/80	All holders of a PWR Power Reactor OL
80-33	Determination of Teletherapy Timer Accuracy	9/15/80	All holders of a teletherapy license
80-32	Clarification of Certain Requirements for Exclusive-use Shipments of Radioactive Materials	9/12/80	All holders of an NRC or Agreement State License
80-31	Maloperation of Gould-Brown Boveri 480V-Type K-600S and K-Dor 600S Circuit Breakers	8/27/80	All holders of a power reactor OL or CP