



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

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In Reply Refer To:
RII:JPO

50-325, 50-324

50-400, 50-401

50-402, 50-403

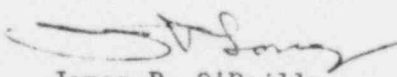
50-261

Carolina Power and Light Company
ATTN: J. A. Jones
Senior Executive Vice President and
Chief Operating Officer
411 Fayetteville Street
Raleigh, NC 27602

Gentlemen:

This Information Notice is provided as an early notification of a possibly significant matter. It is expected that recipients will review the information for possible applicability to their facilities. No specific action or response is requested at this time. If further NRC evaluations so indicate, an IE Circular or Bulletin will be issued to recommend or request specific licensee actions. If you have questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

Sincerely,


James P. O'Reilly
Director

Enclosures:

1. IE Information Notice No. 80-41
2. List of Recently Issued
IE Information Notices

cc w/encl:

R. B. Starkey, Jr., Plant Manager
R. P. rsons, Site Manager
A. C. Tollison, Jr., Plant Manager

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

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November 10, 1980

IE Information Notice No. 80-41: FAILURE OF SWING CHECK VALVE IN THE DECAY
HEAT REMOVAL SYSTEM AT DAVIS-BESSE UNIT
NO. 1

Description of Circumstances:

On October 9, 1980 the resident inspector at the Davis-Besse facility was informed that the licensee had performed leak rate tests and identified excessive leakage through Decay Heat Removal System check valve CF-30. Valve CF-30 is the inboard one of two in-series check valves that is used to isolate the reactor coolant system from the low pressure decay heat removal system. On further investigation the licensee found that the valve disc and arm had separated from the valve body and was lodged just under the valve cover plate. The two 2 5/8 x 5/8 inch bolts and locking mechanism for the bolts that holds the arm to the valve body were missing and have not been located. The CF-30 valve is a 14-inch swing check valve manufactured by Velan Valve Corporation. The cause of the failure has not been identified.

The test that was in progress when the failure was identified was being conducted in response to a letter from Darrell G. Eisenhut, Office of Nuclear Reactor Regulations to all Light Water Reactor Licensees, LWR Primary Coolant System Pressure Isolation Valves, February 23, 1980.

This IE Information Notice is provided as an early notification of a possibly significant matter that is still under review by the NRC staff. It is expected that recipients will review the information for possible applicability to their facilities. No specific action or response is requested at this time. If NRC evaluations so indicate, further licensee actions may be requested or required.

No written response to this IE Information Notice is required. If you have any questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

RECENTLY ISSUED
IE INFORMATION NOTICES

Information Notice No.	Subject	Date of Issue	Issued to
80-41	Failure to Swing Check Valve in the Decay Heat Removal System at Davie-Besse Unit No. 1	11/10/80	All nuclear power reactor facilities with an OL or CP
80-40	Failure Of Swing Check Valve In The Decay Heat Removal System At Davis-Besse Unit No. 1	11/10/80	All power facilities with and OL or CP
80-39	Malfunctions Of Solenoid Valves Manufactured By Valcor Engineering Corporation	10/31/80	All light water reactor facilities holding power reactor OLs or CPs
80-38	Cracking In Charging Pump Casing Cladding	10/30/80	All PWR facilities with an OL or CP
80-37	Containment cooler leaks and reactor cavity flooding at Indian Point Unit 2	10/24/80	All nuclear power facilities holding power reactor OLs or CPs
80-36	Failure of Steam Generator Support Bolting	10/10/80	All nuclear power reactor facilities holding power reactor OLs or CPs
80-35	Leaking and dislodged Iodine-124 implant seeds	10/10/80	All categories G and G1 medical licensees
80-34	Boron dilution of reactor coolant during steam generator decontamination	9/26/80	All pressurized water reactor facilities holding power reactor OLs
80-33	Determination of teletherapy timer accuracy	9/15/80	All teletherapy (G3) licensees
80-32	Clarification of certain requirements for Exclu- sive-use shipments of radioactive materials	3/12/80	All NRC and agreement state licensees
80-31	Maloperation of Gould- Brown Boveri Type 480 volt type K-600S and K-DON 600S circuit breakers	8/27/80	All light water reactor facilities holding OLs or CPs

* Operating Licenses or Construction Permits