DAIRYLAND POWER COOPERATIVE · P.O. BOX 817 · 2615 EAST AV. SOUTH · LA CROSSE, WISCONSIN 54601
(608) 788-4000

November 28, 1980

In reply, please
refer to LAC-7253

DOCKET NO. 50-409

Mr. James G. Keppler
Regional Director
U. S. Nuclear Regulatory Commission
Directorate of Regulatory Operations
Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

SUBJECT: DAIRYLAND POWER COOPERATIVE
LA CROSSE BOILING WATER REACTOR (LACBWR)
PROVISIONAL OPERATING LICENSE NO. DPR-45
REPORTABLE OCCURRENCE NO. 80-14

Reference: (1) LACBWR Technical Specifications, Section 3.9.2.a.(3).
(2) LACBWR Technical Specifications, Section 5.2.1.2.(b)4.

Dear Mr. Keppler:

In accordance with Reference 1, this is to inform you of abnormal degradation discovered in the primary reactor containment during periodic Type C surveillance leakage rate testing while in a shutdown condition.

As a result of conducting periodic Type C leak tests on the Containment Building Heating Steam Condensate Valve, the leak flow rates determined from the Condensate Valve exceeded the permissible leak flow rates as defined in Technical Specifications (Reference 2).

The test leakage rate was determined to be 14.3 SCFH. The Acceptance Criteria is 0.375 SCFH.*

The test was performed on November 27, 1980. Prompt notification of test results to Mr. Bill Little of the U. S. Nuclear Regulatory Commission, Region III, was accomplished at 1000 on November 28, 1980, by L. S. Goodman, LACBWR Operations Engineer.

*Shall not exceed 1% of leakage rate L_{po} where L_{po} is 37.5 SCFH.

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Repairs or adjustments to the Heating Steam Condensate Valve will be accomplished and a retest will be performed.

If there are any questions concerning this report, please contact us.

Very truly yours,

DAIRYLAND POWER COOPERATIVE

Frank Linder

Frank Linder, General Manager

FL:JDP:af

cc: Director, Office of Management Information and Program Control (3)
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

NRC FORM 366
(7-72)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | (2) LICENSEE CODE | (3) LICENSE NUMBER | (4) LICENSE TYPE | (5) CAT

CON'T

0 1 | REPORT SOURCE (6) | (7) DOCKET NUMBER | (8) EVENT DATE | (9) REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2

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0 9 | SYSTEM CODE (11) | CAUSE CODE (12) | SUBCODE (13) | COMPONENT CODE (14) | COMP. SUBCODE (15) | VALVE SUBCODE (16)

(17) LER/RO REPORT NUMBER | EVENT YEAR | SEQUENTIAL REPORT NO. | OCCURRENCE CODE | REPORT TYPE | REVISION NO.

ACTION TAKEN (18) | FUTURE ACTION (19) | EFFECT ON PLANT (20) | SHUTDOWN METHOD (21) | HOURS (22) | ATTACHMENT SUBMITTED (23) | NPRD-4 FORM SUB. (24) | PRIME COMP. SUPPLIER (25) | COMPONENT MANUFACTURER (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0

1 1

1 2

1 3

1 4

1 5 | FACILITY STATUS (28) | % POWER (29) | OTHER STATUS (30) | METHOD OF DISCOVERY (31) | DISCOVERY DESCRIPTION (32)

1 6 | ACTIVITY RELEASED (33) | CONTENT OF RELEASE (34) | AMOUNT OF ACTIVITY (35) | LOCATION OF RELEASE (36)

1 7 | PERSONNEL EXPOSURES NUMBER (37) | TYPE (38) | DESCRIPTION (39)

1 8 | PERSONNEL INJURIES NUMBER (40) | DESCRIPTION (41)

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE (42) | DESCRIPTION (43)

2 0 | PUBLICITY ISSUED (44) | DESCRIPTION (45) | NRC USE ONLY

NAME OF PREPARER _____ PHONE: _____

GPO 917-926