## MORTHEAST UTILITIES



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October 20, 1980

Docket No. 50-336 A00921 O KOZI 10 MI I I I I I

Mr. Boyce H. Grier, Director Region 1 Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, PA 19406

References: (1) B. H. Grier letter to W. G. Counsil dated March 13, 1980, transmitting I&E Bulletin No. 80-06.

(2) W. G. Counsil letter to B. H. Grier dated June 13, 1980.

Gentlemen:

Millstone Nuclear Power Station, Unit No. 2

I&E Bulletin No. 80-06 - Engineered Safety Features Reset Control

In Reference (1), the NRC Staff requested that Northeast Nuclear Energy Company (NNECO) investigate concerns related to the reset logic of engineered safety feature (ESF) components.

NNECO's initial response to Reference (1) was docketed by Reference (2). Included in Reference (2) was a listing of safety-related equipment at Millstone Unit No. 2 and whether that equipment meets the acceptance criteria described in Reference (1). The Reference (2) listing identified thirty three (33) components which have reset logic not in conformance with the recommendations of Reference (1). NNECO committed to modify the circuitry associated with these components by January 1, 1981. In the interim, the emergency operating procedures have been revised to inform operators of the equipment changes that occur upon reset of the actuation signals.

NNECO has since reviewed those components identified by Reference (2) as not meeting the Reference (1) acceptance criteria and has determined that circuit modifications to the reset logic associated with Items 6, 48, 132, 136, 180, and 187 and Reference (2) are not required. These components are described below.

Item	Equipment Description	Equipment Number	ESF Signals/Channel
6	LPSI Pump	P-42A	SIAS/1
48	LPSI Pump	P-42B	SIAS/2
132	LPSI Pump	P-42A	SRAS/1
136	LPSI Pump	P-42B	SRAS/2
180	Steam Generator #1 Feedwater Isolation Valve	HV-5419	MSI/1
187	Steam Generator #2 Feedwater Isolation Valve	HV-5420	MSI/2

Subsequent review of the function of these components has revealed that the response of these components to a reset signal is of no operational concern. Therefore, the circuitry modifications and resultant loss of control panel space are not justified. These components are addressed in the appropriate operating procedures and, as such, adequate control and operator awareness of the status of the equipment is assured. It remains NNECO's intention to complete the modifications associated with the remaining 27 items by January 1, 1981.

We trust you find this information satisfactory.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

W. G. Counsil

Senior Vice President