



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

November 21, 1980

Honorable John F. Ahearne
Chairman
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: TWO-HUNDRED-FORTY-SEVENTH MEETING OF THE ADVISORY COMMITTEE ON REACTOR SAFEGUARDS, NOVEMBER 6-8, 1980

Dear Dr. Ahearne:

The Advisory Committee on Reactor Safeguards held its 247th meeting in Washington, DC on November 6-8, 1980. Items considered at this meeting are summarized below:

The Committee met with Governor Bruce Babbitt, Chairman of the Nuclear Safety Oversight Committee, to discuss methods to strengthen the ACRS' role in the regulatory process.

The Committee met with you and the other Commissioners to discuss clarification of Commissioner Gilinsky's request for ACRS' comments regarding proposed changes in the residual heat removal system of the North Anna Power Station Unit 2, new safety concepts for future nuclear power plant designs, and de facto use of quantitative safety goals in the regulatory process.

The Committee met with representatives of the NRC Staff and the General Electric Company to discuss the seismic modifications and proposed operation of the General Electric Test Reactor (GETR).

The Committee met with members of the NRC Staff and representatives of Pacific Gas and Electric Company to discuss the study of potential systems interactions induced by possible seismic events performed for the Diablo Canyon Nuclear Power Plant.

The Committee continued its consideration of the proposed Nuclear Data Link.

The Committee responded to Commissioner Gilinsky's memorandum of October 2, 1980 regarding the format of findings in ACRS reports on license applications.

The Committee completed its review of Regulatory Guide 1.97, Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident.

Copies of the letters and memoranda pertaining to the above have been provided to you.

The Committee met with members of the NRC Staff to discuss the status of the effort to develop, a system to control combustible gases for the Sequoyah Nuclear Power Plant Unit One. Additional work on this matter is planned for future meetings.

Since the last report of ACRS Activities, the following site visits and subcommittee meetings have been held:

Site Visit to Idaho Nuclear Engineering Laboratory, Idaho Falls, ID on October 21, 1980

ECCS, in Idaho Falls, ID on October 22-23, 1980

Transportation of Radioactive Materials, in Washington, DC on October 27, 1980

General Electric Test Reactor in Washington, DC on November 4, 1980

Regulatory Activities, in Washington, DC on November 5, 1980

Procedures and Administration, in Washington, DC on November 5, 1980

The following items are tentatively scheduled for consideration during the 248th ACRS Meeting, December 4-6, 1980.

- Three Mile Island Nuclear Station Unit 1 -- Restart
- Regulatory Guides regarding Utilities Management Structure and Technical Support
- North Anna Power Station Unit 2 -- Consider installation of a heat exchanger in the High Pressure Injection System and environmental qualification of the Residual Heat Removal System, Plant Staffing, results of Augmented Low Power Test Program, Steam Generator Tube Integrity, and Probabilistic Assessment of the Plant
- Clarification of ACRS Report on ATWS per GE Request
- ACRS Generic Items List -- Update of list
- Hydraulic Scram System for BWR Nuclear Plants
- New Safety Concepts for future reactor designs
- NRC Waste Confidence Proceeding -- Items needing further consideration

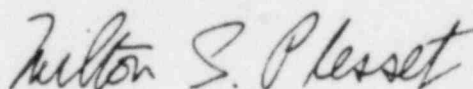
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- Indian Point Station Unit 2 -- Cavity flooding incident/plant status
- Meeting with NRC Commissioners

ACRS Review of Procedures for certification of shipping containers for radioactive materials.

ACRS Review of Long Range Safety Research Program.

Sincerely yours,

A handwritten signature in cursive script that reads "Milton S. Plesset".

Milton S. Plesset
Chairman