UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT Docket No. 50-282
50-306

REQUEST FOR AMENDMENT TO
OPERATING LICENSE NO. DPR-42 & DPR-60

(License Amendment Request Dated November 24, 1980)

Northern States Power Company, a Minnesota corporation, requests authorization for changes to the Technical Specifications as shown on the attachments labeled Exhibit A and Exhibit B. Exhibit A describes the proposed changes along with reasons for the change. Exhibit B is a set of Technical Specification pages incorporating the proposed changes.

This request contains no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By L. O. Mayer
L O Mayer
Manager of Nuclear Support Services

On this 24th day of November, 1980, before me a notary public in and for said County, personally appeared L O Mayer, Manager of Nuclear Support Services, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof and that to the best of his knowledge, information and belief, the statements made in it are true and that it is not interposed for delay.

Jeanne M Hacker

JEANNE M. HACKER
NOTARY PUBLIC - MINNESOTA
HENNEPIN COUNTY
My Commission Expires May 6, 1986

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EXHIBIT A
Prairie Island Nuclear Generating Plant

License Amendment Request dated November 24, 1980

Proposed Changes to the Technical Specifications
Appendix A of Operating Licenses
DPR-42 and DPR-60

Pursuant to 10 CFR 50.59, the holders of Operating Licenses DPR-42 and DPR-60 hereby propose the following changes to Appendix A, Technical Specifications:

1. Decay Heat Removal Capability

PROPOSED CHANGE

Revise the Technical Specifications to require at least two means of decay heat removal to be operable at all times.

Above 350 degrees F, revise the Limiting Conditions for Operation to require both reactor coolant loops to be operable at all times.

Below 350 degrees F, require at least two redundant means of decay heat removal to be operable. Either an RHR loop or a reactor coolant loop is an acceptable means of decay heat removal. With the reactor coolant system drained, require both RHR loops to be operable.

Refer to Exhibit B, pages TS.3.1-1, TS.3.1-1A, and TS.3.1-3.

REASON FOR CHANGE

These changes are being submitted at the request of the NRC Staff. The need for these additional requirements became evident following the Davis-Besse Unit No. 1 event earlier this year in which decay heat removal capability was not assured.

The proposed changes follow the recommendations contained in a letter dated June 11, 1980, from D G Eisenhut, Director, Division of Licensing, USNRC, with one exception. Credit is taken for natural circulation decay heat removal capability in our submittal.

SAFETY EVALUATION

These changes add additional requirements to the Technical Specifications related to decay heat removal systems. The proposed changes are based on guidance provided by the NRC Staff. Existing requirements of the Technical Specifications are not affected by these changes.

2. Water Level Over Top of Irradiated Fuel Assemblies

PROPOSED CHANGE

Revise the Technical Specifications to require at least 23 feet of water over the reactor vessel flange at all times during movement of fuel assemblies or control rods. Add a surveillance requirement for measuring this water level on a daily basis.

Revise the Technical Specifications to require both RHR loops to be operable during refueling operations if there is less than 20 feet of water above the top of the reactor vessel flange.

Refer to Exhibit B, pages TS.3.8-1, TS.3.8-4, and Table TS.4.1-2A.

REASON FOR CHANGE

These changes add Limiting Conditions for Operation and Surveillance Requirements related to water level in the reactor cavity during refueling operations. They provide additional assurance that there is sufficient water in the cavity to prevent exposure of irradiated fuel and control rods during movement of these components. In addition, in the event water level falls below normal, they provide additional assurance that redundant means for removing decay heat are available.

The proposed changes follow the recommendations contained in letters dated June 11, 1980 from D G Eisenhut, Director, Division of Licensing, USNRC, and August 15, 1980 from T M Novak, Assistant Director for Operating Reactors, Division of Licensing, USNRC, with one exception. A cavity water level of 20 feet has been proposed as the minimum required to provide decay heat removal capability. A 23-foot requirement at our facility would be impractical to impose except during actual fuel movements. A level of 20 feet provides a sufficiently large water volume for decay heat removal purposes.

SAFETY EVALUATION

These changes add additional requirements to the Technical Specifications related to preventing exposure of irradiated fuel assemblies and control rods during handling and assurance of redundant means for removing decay heat during refueling operations. The proposed changes are based on guidance provided by the NRC Staff. Existing requirements of the Technical Specifications are not affected by these changes.

Note: As shown in Exhibit B, Page TS.3.8-1, reference to the minimum boron concentration for cycle 1 in Specification 3.8.A.5 can be conveniently deleted in conjunction with these changes. In addition, minor grammatical errors in Specifications 3.8.A.1 and 3.8.A.3 are corrected.