



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report Nos. 50-425/80-12 and 50-425/80-12

Licensee: Georgia Power Company
270 Peachtree Street
Atlanta, Georgia 30303

Facility Name: Plant Vogtle Units 1 and 2

License Nos. CPPR-108 and CPPR-109

Inspection at: Vogtle site near Waynesboro, Georgia

Inspector: M. D. Hunt

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10/8/80
Date Signed

Approved by: J. C. Bryant

J. C. Bryant, Section Chief, RCES Branch

11/5/80
Date Signed

SUMMARY

Inspection on September 9-11, 1980

Areas Inspected

This routine, unannounced inspection involved 16 inspector-hours onsite in the areas of licensee identified items, IE Bulletins, inspector followup items, QA audits, and field drawing control.

Results

Of the five areas inspected, no items of noncompliance or deviations were identified.

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DETAILS

1. Persons Contacted

- *K. M. Gillespie, Construction Project Manager
- H. H. Gregory, Assistant Construction Project Manager
- *C. R. Miles, Jr., QA Field Supervisor
- *E. D. Groover, QA Site Supervisor
- *B. C. Harbin, Project Engineering Section Supervisor
- *D. M. Fiquett, Manager, Field Operations
- *M. H. Googe, Manager, QA

Other licensee employees contacted included 5 construction craftsmen, 4 technicians, and 3 office personnel.

Other Organizations

- *J. E. Mahlmeister, Resident Engineer, Bechtel Power Corporation (BPC)
- D. P. Armstrong, Civil Resident Engineer, BPC
- *F. R. McCarty, Project Manager, Walsh Construction Company (WCC)
- *G. Ryan, QA, WCC

*Attended exit interview

2. Exit interview

The inspection scope and findings were summarized on September 11, 1980 with those persons indicated in Paragraph 1 above.

3. Licensee Action on Previous Inspection Findings

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Independent Inspection Effort

- a. QA Audit GD05-80/26 - The inspector reviewed the subject QA audit performed June 10 through July 31, 1980 entitled "Nonconformance Control". The audit investigated the processing of nonconforming reports (NCR) for clarity of the problem description, proper and prompt disposition of the problem and proper closure of the NCR. The audit contained 7 open items, one against the program and two each against the civil, mechanical and electrical engineering branches. The inspector reviewed the responses submitted by each of the engineering disciplines. Each appeared to adequately address the problems cited.

- b. Field Drawing Review - The inspector made a random selection of 13 drawings in use by the crafts in the field. These drawings were examined to determine that they were current. In all cases the drawings were at the correct revision level and appropriate approved field drawing change requests were attached to the drawings.

Within the areas examined, no items of noncompliance or deviation were identified.

6. Licensee Identified Items (LII) 10 CFR50.55(e)

(Closed) LII 424/78-09-02 and 425/78-09-03 Improper Fabrication of Embeds. The licensee has evaluated each embed that was fabricated and installed without proper controls. Some will be abandoned and others have been derated, but all are identified and the status of each embed indicated.

(Open) LII 424/425/80-12-01, Holddown Bolting Material - Southern Bolt & Fastener Company. GPC was informed that bolt material furnished by Southern Bolt & Fastener Company as part of Heat #69784 may not meet the required tensile strength. The AE is investigating.

(Open) LII 424/80-12-02 - Collapse of Cooling Tower 1A Rebar Steel. The licensee reported that approximately one third of the inner ring of vertical rebar extending 30 feet above the concrete construction joint collapsed when a supporting ring was removed in nuclear service water cooling tower 1A. In addition to the bent rebar, the concrete wall and base slab were damaged. The licensee has started evaluating the extent of the damage and the cause.

(Open) LII 424/80-12-03, Understrength Concrete Used in Reactor Building. The licensee reported that an eight (8) yard batch of 2000 lb concrete was inadvertently placed in a 240 yard (5000 lb) placement in the access shaft to the tendon gallery in Unit 1 reactor building. The licensee has taken steps to better identify the concrete delivery trucks. This placement was partially removed with water. The remainder will be chipped out and the entire placement will be replaced.

7. Inspector Followup Items (IFI)

(Closed) IFI 424/425/80-10-02, Receipt Inspection Interface. The licensee has revised Procedure GD-A-30 to require the notification of all involved disciplines when equipment is received that requires inspection.

(Closed) IFI 424/425/80-02-02, Instruction and Training for Control of Cold Spring in Piping. Pullman Power Products (PPP) has revised procedure IX-19 to include indoctrination of each welder in cold spring limitations. Each welder is required to acknowledge that the training was given.

(Closed) IFI 424/425/79-17-02, Reactor and Auxiliary Building Sump Capacity.

(Closed) IFI 424/425/79-17-03, Radwaste Storage and Treatment Systems Capacities. These two items are to be addressed in the FSAR. The licensee presented a letter from his AE which states that the subject inspector concerns will be addressed.

8. IE Bulletins (IEB)

(Closed) IEB 80-05. The licensee's response dated August, 1980 addressed the subject bulletin adequately in each requirement.

(Closed) IEB 80-09. The licensee's response dated June 16, 1980 stated that GPC will not use AH-90 and NH-90 Hydro motor actuators in any safety related systems at Plant Vogtle.

(Closed) IEB 80-10, 80-11, 80-12. These bulletins were issued for information only due to the construction status of the plant.

(Closed) IEB 80-16. The licensee's September 2, 1980 response to this IEB states that Rosemont Models 1151 and 1152 pressure transmitters and Rosemont Model 510DU trip units will not be used in safety related systems at Plant Vogtle.

(Closed) IEB 80-19. The licensee's September 2, 1980 response advises that no mercury-wetted relays for matrix logic applications are provided by the AE or NSSS supplier. The only relays of this type are used for temperature channel test inputs, are hermetically sealed and are not used in reactor trip circuits.