CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 0 CON'T 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) [Unit 1 in Mode 4 with RCS Tave at 207 degrees F and pressure at 325 psig, the lower 0 2 jairlock failed SI-159.1, overall leakage test. Allowable leakage for the airlock 03 is < = 11.25 SCFH. The airlock leakage was 39.6 SCFH. The action statement of LCO 0 4 13.6.1.3 was entered. The Technical Specification requirement for overall containment 0 5 (integrity (bypass and summation leakage) was not exceeded. There was no effect upon 0 6 public health or safety. Previous occurrences - none. 0 7 SYSTEM CAUSE CAUSE COMP CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE B (13) NI TR 019 OCCURRENCE SEQUENTIAL REVISION REPOR REPORT NO. 2005 LER/RO HEPORT 11716 013 0 11. NUMBER SUBMITTED NPRD 4 PRIME COMP SHUTDOWN COMPONENT HOURS (22) FORM SUS. METHOD SUPPLIER MANUFACTURER N CA 101 01 Z (21 0 01 C 13 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The source of the leakage was the airlock inner door ball valve. The valve was 1101 repaired and retested. A small leak around the outer door handwheel seal was discovered during subsequent retesting. The seal was tightened and the airlock was satisfactorily retested. The corrective action was performed within the allowable 24 hour time limit of LCO 3.6.1.3 action statement. 80 B (3) Surveillance testing STATE OTHER STATUS (30) 3 POWER DISCOVERY DESCRIPTION (32) 01010 (0) G (28) NA 13 80 CONTENT AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) NA (33) (34) NA 80 PERSONNEL EXPOSURES DESCRIPTION (30 NUMBER TYPE 0 0 0 80 PERSONNEL INJURIES DESCRIPTION(41) NUMBER 0 0 0 (40) NA 8 OSS OF OR DAMAGE TO FACILITY (4) 80 DESCRIPTION NA 9 2 1(42) 80 SSUED DESCRIPTION 45 8011250339 NRC USE ONLY 210 68 69 Phone 615-842-8261 Name of Preparer W. T. Cottle/G. B. Kirk

EVALUATION LOGIC FOR PART 21

		Originating	Document Yes	No. <u>LER-SQRO-</u> 50-327/80176
Ι.	Deficiency of a plant security system?		_	<u></u>
	1. Could defect create a substantial safety	hazard?		<u>_X</u>
	If yes, report as part 21.			
	Is the component necessary to ensure:		Υ.	
	1. The integrity of the reactor coolant bound	ndary?	_	<u>_X</u>
	 The capability to shut down reactor and a it in a safe shutdown condition? 	maintain		<u>_X</u>
	3. The capability to prevent or mitigate the consequences of accidents which could response to comparable to referred to in 10 CFR 100.11?	sult in those		<u>_x</u>
	Is defect in a basic component one that has be accepted for ownership?	en	_	<u></u>
	Installed for use or operation?			<u></u>
	If a yes in II and III above, could defect cre a substantial safety hazard?	ate	-	<u></u>
	If yes, report as part 21.			
IV.	Is defect in a basic component:			
	A condition that could contribute to exceeding of safety limit?		1	<u></u>
If ye	es to one of II and IV above, report as part 21			