

Public Service Company of Colorado

16805 ROAD 19½ PLATTEVILLE, COLORADO 80651

> August 5, 1980 Fort St. Vrain Unit No. 1 P-80246

Mr. Karl V. Seyfrit, Director Nuclear Regulatory Commission Region IV Office of Inspection and Enforcement 611 Ryan Plaza Drive Suite 1000 Arlington, Texas 76012

> Reference: Facility Operating License No. DPR-34

> > Docket No. 50-267

Dear Mr. Seyfrit:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/ 80-36, Final, submitted per the requirements of Technical Specification AC 7.5.2(b)2.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/80-36.

Very truly yours,

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Don Warembourg Manager, Nuclear Production

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Enclosure

cc: Director, MIPC

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OCCURRENCE REPORT DISTRIBUTION

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Mr. Karl V. Seyfrit, Director	0	(Original of P Letter and Copy of LER)	
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W. Bushneil, Site Manager General Atomic Company P. O. Box 426 Platteville, Colorado 80651	10	(Original of FPLG Letter plus Two Copies, One Copy of P Letter, One Copy of LER)	
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FORT ST. VRAIN NUCLEAR GENERATING STATION PUBLIC SERVICE COMPANY OF COLORADO 16805 WELD COUNTY ROAD 19 1/2 PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/80-36/03-L-0

Final

IDENTIFICATION OF OCCURRENCE:

Between July 6, 1980, and July 8, 1980, with the reactor operating at power, the total primary coolant oxidants exceeded the limits of LCO 4.2.10, and the reactor dewpoint exceeded the limits of LCO 4.2.11.

This constitutes operation in a degraded mode of LCO 4.2.10 and LCO 4.2.11, and is reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)2.

EVENT DESCRIPTION:

Following the June 17 loop shutdown and turbine trip, preparations were made for the return to power, and the turbine generator was placed on line at 0626 hours, June 30, 1980. Reactor power was increased to about 45% until excess moisture required a reduction of power to be in compliance with LCO 4.2.11 (See RO 50-267/80-33/03-L-0).

Following this power reduction, power was increased while keeping the average core outlet temperature low enough to avoid exceeding the allowable reactor dewpoint of LCO 4.2.11.

This continued until July 6, 1980, at 1600 hours where primary coolant moisture again began increasing and the reactor dewpoint went into the limited acceptable area of Figure 4.2.11-1. Reactor power was reduced to prevent exceeding the allowable reactor dewpoint. By 2400 hours on June 6, 1980, the reactor dewpoint was again in the acceptable area of Figure 4.2.11-1.

On June 7, 1980, reactor power was again increased while maintaining reactor dewpoint within the limits of Figure 4.2.11-1. This continued till approximately 0930 hours when the System Control Center asked for more power to compensate for the losses of a unit and some purchased power. Reactor power was increased to 55% and the average core outlet temperature exceeded 1200°F. With total primary oxidants greater than 10 ppm, this resulted in operation in a degraded mode of LCO 4.2.10. This continued until approximately 1830 hours when the System Control Center notified the plant that the day's plak demand was over and power reduction could begin. At approximately 2000 hours, average core outlet temperature dropped below 1200°F, ending the period of operation in a degraded mode of LCO 4.2.10.

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EVENT DESCRIPTION: (Cont'd)

The reactor dewpoint at this time was higher than allowed by Figure 4.2.11-1 and a degraded mode of LCO 4.2.11 was entered. This continued as reactor power was decreased until 0837 hours on July 8, 1980, when a reactor scram occurred.

CAUSE DESCRIPTION:

The increase in primary coolant moisture and reactor dewpoint during this period is being attributed to residual moisture from the June 17 circulator trips and a cross-tower leakage problem on the buffer helium dryer due to HV-21352 leakage. This leakage was identified on July 7, 1980, and HV-21352 was disassembled and repaired on July 8, 1980.

CORRECTIVE ACTION:

Reactor power was reduced, resulting in lower average core outlet temperatures and operation in accordance with LCO 4.2.10 and LCO 4.2.11.

The buffer helium dryer valve HV-21352 was repaired to reduce the cross-tower leakage and resulting moisture ingress.

No further corrective action is anticipated or required.

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