OYSTER CREEK **NUCLEAR GENERATING STATION**

JCP&L GPU

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October 14, 1980

Mr. Boyce H. Grier, Director
Office of Inspection and Enforcement
Region I
United States Nuclear Regulatory Commission
641 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

SUBJECT: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report
Reportable Occurrence No. 50-219/80-43/3L

This letter forwards three copies of a Licensee Event Report to report Reportable Occurrence No. 50-219/80-43/3L in compliance with paragraph 6.9.2.b.1 of the Technical Specifications.

Very truly yours,

Ivan R. Finfrock, Jr.
Vice President Generation

IRF:dh
Enclosures

cc: Mr. John G. Davis, Acting Director (40 copies)
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. William G. McDonald, Director (3 copies)
Office of Management Information and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731

License Event Report
Reportable Occurrence No. 50-219/80-43/3L

Report Date

October 3, 1980

Occurrence Date

September 25, 1980

Identification of Occurrence

Exceeding a limiting condition for operation as per Technical Specifications, Section 3.1, Table 3.1.1, Function G.2, when reactor triple low water level sensor RE18D exceeded its required setpoint during surveillance testing.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.b.1.

Conditions Prior to Occurrence

Steady State Power

Power:	Reactor	1913 MWt
	Generator	635 MWe
Flow:	Recirculation	15.4×10^4 gpm
	Feedwater	7.1×10^6 lb/hr

Description of Occurrence:

On Thursday, September 25, 1980, at approximately 1110 hours, while performing routine surveillance testing of the reactor triple low water level sensors, RE18D tripped at a level which was less conservative than that specified in the Technical specifications.

Tests on all level sensors yielded the following data:

Pressure Switch Designation		Desired Manometer Reading at Trip Point ("H_2O)	As Found ("H_2O)	As Left ("H_2O)
System I	RE18A	$\leq$126	124.5	122.1
	RE18C	$\leq$126	123.5	122.5
System II	RE18B	$\leq$126	125.8	122.0
	RE18D	$\leq$126	128.0	121.5

The "As Found" value of 128.0" H_2O corresponds to a water level 54" abcve the active fuel. The Technical Specification limit is 56".

Apparent Cause of Occurrence

Sensor Repeatability

Analysis of Occurrence

Failure of pressure switch RE18D to actuate at its prescribed setpoint would have delayed initiation of reactor triple low water level indications. However, due to the existing logic configuration, the redundant switch, RE18B, would have actuated to initiate the required functions at the required Technical Specification limit. The safety significance of this event is considered to be minimal since sensor RE18D's non-conservatism resulted only in a temporary loss of redundancy in the system.

Corrective Action

Reactor triple low level sensor RE18D was reset to trip within its prescribed limits.

An engineering study is in progress regarding the feasibility of replacing the existing sensors with a solid state system. Additionally, an evaluation will be performed by the manufacturer(ITT Barton) on the drift problem experienced with the snap-action switches.

Failure Data

ITT Barton Differential Pressure Indicating Switch
Switch Model #288A
Adjustable Range 0-150 inches H_2O