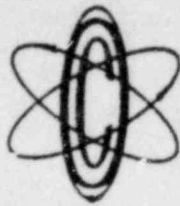


**OYSTER CREEK**



**NUCLEAR GENERATING STATION**

**JCP&L GPU**

Jersey Central Power & Light  
Company is a Member of the  
General Public Utilities System

(609) 693-1951 P.O. BOX 388 • FORKED RIVER • NEW JERSEY • 08731

October 14, 1980

Mr. Boyce H. Grier, Director  
Office of Inspection and Enforcement  
Region I  
United States Nuclear Regulatory Commission  
641 Park Avenue  
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

SUBJECT: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Licensee Event Report  
Reportable Occurrence No. 50-219/80-43/3L

This letter forwards three copies of a Licensee Event Report to report Reportable Occurrence No. 50-219/80-43/3L in compliance with paragraph 6.9.2.b.1 of the Technical Specifications.

Very truly yours,

Ivan R. Finrock, Jr.  
Vice President Generation

IRF:dh  
Enclosures

cc: Mr. John G. Davis, Acting Director (40 copies)  
Office of Inspection and Enforcement  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

Mr. William G. McDonald, Director (3 copies)  
Office of Management Information and Program Control  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

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OYSTER CREEK NUCLEAR GENERATING STATION  
Forked River, New Jersey 08731

License Event Report  
Reportable Occurrence No. 50-219/80-43/3L

Report Date

October 3, 1980

Occurrence Date

September 25, 1980

Identification of Occurrence

Exceeding a limiting condition for operation as per Technical Specifications, Section 3.1, Table 3.1.1, Function G.2, when reactor triple low water level sensor RE18D exceeded its required setpoint during surveillance testing.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.b.1.

Conditions Prior to Occurrence

Steady State Power

Power: Reactor 1913 MWt  
Generator 635 MWe

Flow: Recirculation  $15.4 \times 10^4$  gpm  
Feedwater  $7.1 \times 10^6$  lb/hr

Description of Occurrence:

On Thursday, September 25, 1980, at approximately 1110 hours, while performing routine surveillance testing of the reactor triple low water level sensors, RE18D tripped at a level which was less conservative than that specified in the Technical specifications.

Tests on all level sensors yielded the following data:

Pressure Switch Designation	Desired Manometer Reading at Trip Point ("H <sub>2</sub> O)	As Found ("H <sub>2</sub> O)	As Left ("H <sub>2</sub> O)
System I RE18A	<126	124.5	122.1
RE18C	<126	123.5	122.5
System II RE18B	<126	125.8	122.0
RE18D	<126	128.0	121.5

The "As Found" value of 128.0" H<sub>2</sub>O corresponds to a water level 54" above the active fuel. The Technical Specification limit is 56".

Apparent Cause of Occurrence

Sensor Repeatability

Analysis of Occurrence

Failure of pressure switch RE18D to actuate at its prescribed setpoint would have delayed initiation of reactor triple low water level indications. However, due to the existing logic configuration, the redundant switch, RE18B, would have actuated to initiate the required functions at the required Technical Specification limit. The safety significance of this event is considered to be minimal since sensor RE18D's non-conservatism resulted only in a temporary loss of redundancy in the system.

Corrective Action

Reactor triple low level sensor RE18D was reset to trip within its prescribed limits.

An engineering study is in progress regarding the feasibility of replacing the existing sensors with a solid state system. Additionally, an evaluation will be performed by the manufacturer (ITT Barton) on the drift problem experienced with the snap-action switches.

Failure Data

ITT Barton Differential Pressure Indicating Switch  
Switch Model #288A  
Adjustable Range 0-150 inches H<sub>2</sub>O