

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-I
 DATE October 15, 1980
 COMPLETED BY D. G. Mitchell
 TELEPHONE (717) 948-8553

MONTH September 1980

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE October 15, 1980
 COMPLETED BY D. G. Mitchell
 TELEPHONE (717) 948-8553

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: September 1980
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>720.</u>	<u>6575.</u>	<u>53304.</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>0.0</u>	<u>31731.8</u>
13. Reactor Reserve Shutdown Hours	<u>0.</u>	<u>0.</u>	<u>839.5</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>0.0</u>	<u>31180.9</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0.0</u>	<u>0.0</u>	<u>76531071.</u>
17. Gross Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>25484330.</u>
18. Net Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>23840053.</u>
19. Unit Service Factor	<u>0.0</u>	<u>0.0</u>	<u>58.5</u>
20. Unit Availability Factor	<u>0.0</u>	<u>0.0</u>	<u>58.5</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>0.0</u>	<u>56.8</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>0.0</u>	<u>54.6</u>
23. Unit Forced Outage Rate	<u>100.0</u>	<u>100.0</u>	<u>32.1</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE 10-15-80
 COMPLETED BY D. G. Mitchell
 TELEPHONE (717) 948-8553

REPORT MONTH September

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	9-1-80	F	720	D	1				Regulatory Restraint Order

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

OPERATING SUMMARY

The Unit was shutdown the entire month as a result of 3/28/79, Unit 2 accident. Core cooling was provided by the Decay Heat Removal System.

MAJOR SAFETY RELATED MAINTENANCE

While the Unit was in cold shutdown condition, the following major maintenance item was performed.

Diesel Generator "A" (EG-Y-1A) repair and preventive maintenance items were completed. This included inspection and repair of generator bearing, compressor overhaul, instrument calibration, oil leak repairs, and exhaust manifold leak repairs. All associated work was performed with satisfactory test results and while Emergency Diesel Generator "B" remained operable.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).