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Docket No. 50-364

JUL 25 1980

MEMORANDUM FOR: Robert L. Tedesco, Assistant Director for Licensing,
Division of Licensing

FROM: Voss A. Moore, Acting Deputy Director, Division of
Human Factors Safety

SUBJECT: FARLEY, NUCLEAR PLANT UNIT NO. 2

Attached is the Division of Human Factors Safety SER for the TMI-2-related
requirements for Emergency Operating Procedures, I.C.1 and I.C.8, for the
Farley Unit 2 5% license.

Original signed by

Voss A. Moore, Acting Deputy Director
Division of Human Factors Safety

Attachment:
As stated

cc w/attachment:
S. Hanauer
D. Ziemann
D. Wigginton
D. Fischer

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DATE	7/23/80	7/23/80	7/24/80	7/25/80		

SAFETY EVALUATION REPORT SUPPLEMENT
FARLEY NUCLEAR PLANT UNIT NO. 2
REVIEW OF EMERGENCY PROCEDURES

INTRODUCTION

The objective of item I.C of the Task Action Plan (NUREG-0660) is to improve the quality of plant operating procedures to provide greater assurance that operator and staff actions are technically correct, explicit and easily understood for normal, transient, and accident conditions. By letter dated June 30, 1980, the licensee submitted procedures for loss of coolant accident (including small breaks), inadequate core cooling, anticipated transients without trip, steam generator tube rupture, and loss of main feedwater in response to item I.C.8 of the Task Action Plan.

REVIEW OF EMERGENCY PROCEDURES

The procedures submitted by the licensee have been reviewed and are generally consistent with the guidelines for Westinghouse plants. There are a number of minor inconsistencies with specific details of the guidelines and some instructions to the operator are vague. These matters are being discussed with the licensee. After our detailed comments on the procedures are transmitted to the licensee, we will meet with the licensee to discuss procedure revisions required for technical and sequential adequacy. Selected emergency procedures will be walked through a simulator and the plant prior to issuing a license for operation above 5% of rated power.

As we stated above, the selected procedures in their current state are generally consistent with the guidelines for Westinghouse plants. These procedures are in place at the plant and are available for any emergency. Since the procedures deal primarily with the cooldown of the reactor and steam cycle and since the decay heat load at 5% of rated power is minimal, we find the procedures in their current state to be acceptable to support operation up to 5% power for low power testing and training.