



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
631 PARK AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

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Docket Nos. 50-245
50-336
50-423

August 7, 1980

Northeast Nuclear Energy Company
ATTN: Mr. W. G. Council
Senior Vice President - Nuclear
Engineering and Operations
P. O. Box 270
Hartford, Connecticut 06101

Gentlemen:

The enclosed IE Information Notice No. 80-29, "Broken Studs on Terry Turbine Steam Inlet Flange," is forwarded to you for information. No written response is required. If you desire additional information regarding this matter, please contact this office.

Sincerely,

Boyce H. Grier
Boyce H. Grier
Director

Enclosures:

1. IE Information Notice No. 80-29
2. List of Recently Issued IE Information Notices

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cc w/encls:

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ENCLOSURE 1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

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8006190028

IE Information Notice No. 80-29
Date: August 7, 1980
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BROKEN STUDS ON TERRY TURBINE STEAM INLET FLANGE

When removing the governor and stop valve on the Unit 1 steam driven emergency feedwater pump at Arkansas Nuclear One for repair of a steam leak at the steam inlet flange, Arkansas Power and Light discovered that five of the eight studs securing the flange were broken.

The cause of the stud failure is unknown at this time. Metallurgical evaluation of the failed bolting will be performed to identify the mode of failure.

The failed studs are 3/4 in. diameter by 3-1/2 in. long and are thought to be of ASTM-A-193 grade B7 steel. The turbine flange bolting is generally covered with insulation and not visible for inspection. From the information available, the bolting has not been removed or inspected since installation seven to eight years ago.

The steam driven emergency feedwater pump turbine at ANO-1 is a type G turbine manufactured by the Terry Steam Turbine Company of Hartford, Connecticut. The turbine is rated at 680 BHP and 3560 RPM. The turbine operates at a reduced steam pressure of 270 psig and temperature of 400°F and has previously experienced overspeed trips and vibrations which may have been caused by slugs of water from the piping.

Licensees are encouraged to carefully examine insulation in the flange to turbine casing region for evidence of leakage and consider inspection of the turbine steam inlet flange bolting. Further, during surveillance testing, care should be taken to observe if abnormal vibration or other transients occur which could promote loss of bolting integrity.

This Information Notice is provided as an early notification of a possibly significant matter that is still under review by the NRC staff. It is expected that recipients will review the information for possible applicability to their facility. No specific action or response is requested at this time. If you have any questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

RECENTLY ISSUED
IE INFORMATION NOTICES

Information Notice No.	Subject	Date Issued	Issued To
Supplement to 80-06	Notification of Significant Events at Operating Power Reactor Facilities	7/29/80	All holders of power reactor OLs and near term OL applicants
80-28	Prompt Reporting Of Required Information To NRC	6/13/80	All applicants for and holders of power reactor construction permits
80-27	Degradation of Reactor Coolant Pump Studs	6/11/80	All holders of pressurized water power reactor OLs or CPs
80-26	Evaluation of Contractor QA Programs	6/10/80	All holders of a Part 50 License
80-25	Transportation of Pyrophoric Uranium	5/30/80	Material Licensees in Priority/Categories II-A, II-D, III-I and IV-DI; Agreement State Licensees in equivalent categories
80-24	Low Level Radioactive Waste Burial Criteria	5/30/80	All holders of an NRC or NRC Agreement State License
80-23	Loss of Suction to Emergency Feedwater Pumps	5/29/80	All holders of power reactor OLs or CPs
80-22	Breakdown In Contamination Control Programs	5/28/80	All holders of power reactor OLs and near term OL applicants
80-21	Anchorage and Support of Safety-Related Electrical Equipment	5/16/80	All holders of power reactor OLs or CPs
80-20	Loss of Decay Heat Removal Capability at Davis-Besse Unit 1 While in a Refueling Mode	5/8/80	All holders of power reactor OLs or CPs