



Commonwealth Edison
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February 10, 1981

Mr. B. J. Youngblood, Chief
Licensing Branch 1
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Subject: LaSalle County Station
Units 1 and 2 Supplemental
Information In Response to
Q111.86
NRC Docket Nos. 50-373/374
LOD 81-40-16

Dear Mr. Youngblood:

Attached for your review are Piping and Instrumentation drawings associated with the systems delineated in Q111.86 as requiring local valve leak rate testing to assure reactor coolant pressure boundary integrity. Also attached are schematics of the isolation valve details which have been marked to indicate piping system class and key design parameters. The schematics are judged to provide all the necessary information necessary for the NRC Staff to complete its review of Q111.86. Therefore, we request that you return the printed P&ID's to the GE Bethesda office upon completion of your review because those drawings are file copies. In the event you require formal documentation of the P&ID's, copies will be transmitted at your request.

If you have any further questions in this regard, please direct them to this office.

Very truly yours,

L. O. DelGeorge
Nuclear Licensing
Administrator

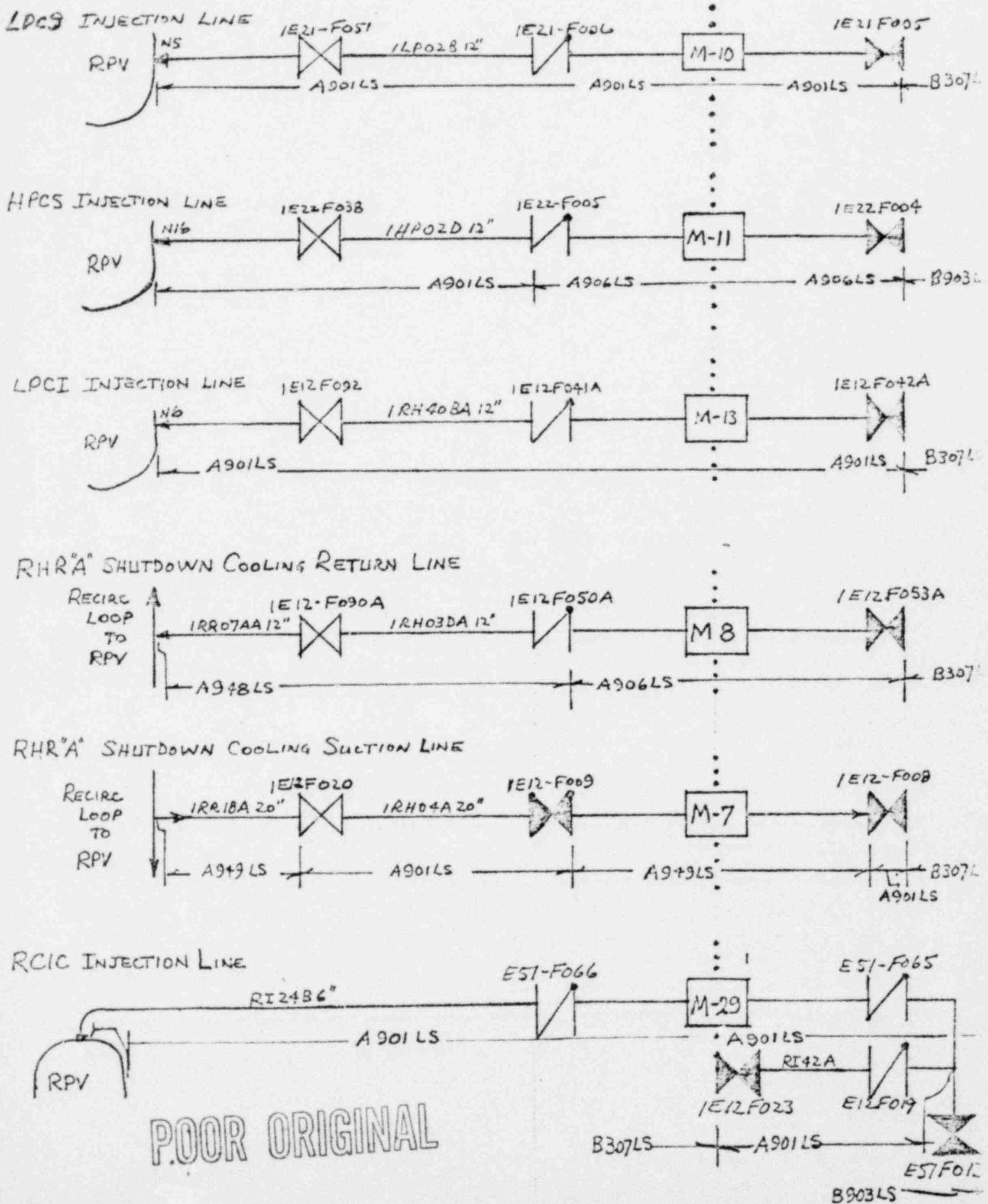
Attachment
cc: NRC Resident Inspector-LSCS

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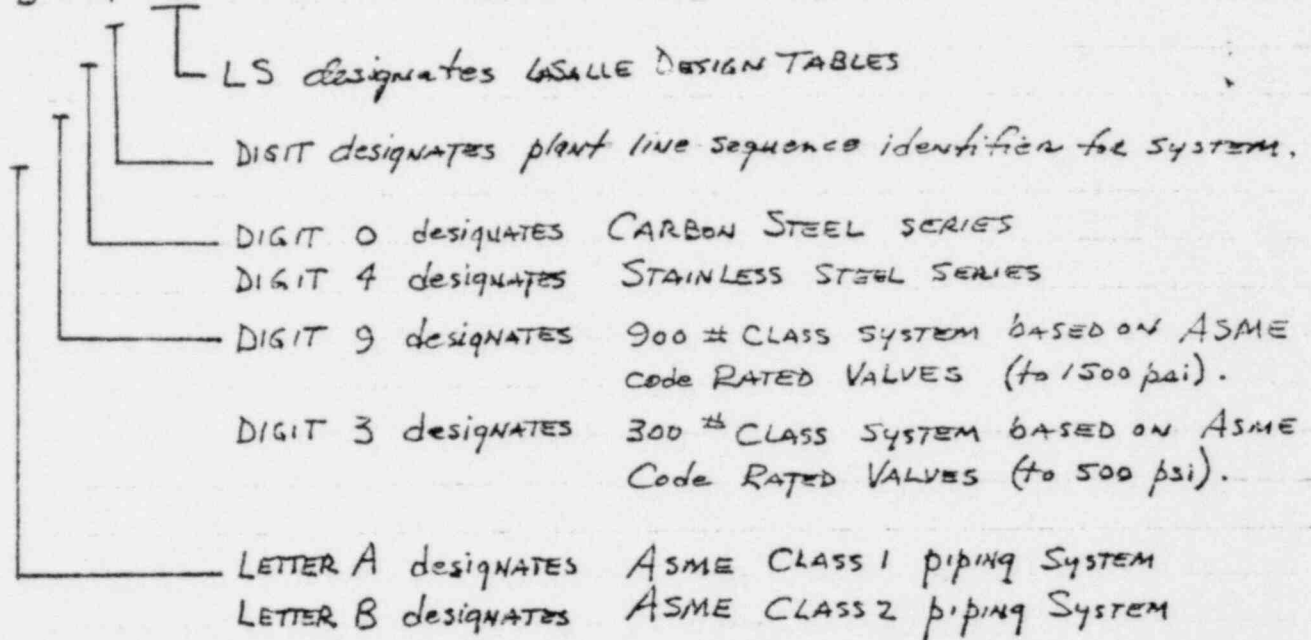
LASALLE UNITS 1 & 2 REACTOR COOLANT SYSTEM OPERATIONAL LEAKAGE



LA SALLE PIPING DESIGN TABLES

A 901 LS
A 906 LS
A 948 LS
A 949 LS

B 903 LS
B 307 LS



CONCLUSION -

FOR PRIMARY PRESSURE CONTAINMENT BOUNDARY, OUT TO AND INCLUDING THE OUTER ISOLATION VALVE, THE LA SALLE ECCS PIPING DESIGN IS ADEQUATE FOR REACTOR DESIGN PRESSURE (1250 psi) PLUS MARGIN TO APPROXIMATELY 1500 psi.

POOR ORIGINAL