

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE February 15, 1981
 COMPLETED BY D. G. Mitchell
 TELEPHONE (717) 948-8553

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit 1
2. Reporting Period: January 1981
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.</u>	<u>744.</u>	<u>56257.</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>0.0</u>	<u>31731.8</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>839.5</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>0.0</u>	<u>31180.9</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0.0</u>	<u>0.0</u>	<u>76531071.</u>
17. Gross Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>25484330.</u>
18. Net Electrical Energy Generated (MWH)	<u>0.</u>	<u>0.</u>	<u>23840053.</u>
19. Unit Service Factor	<u>0.0</u>	<u>0.0</u>	<u>55.4</u>
20. Unit Availability Factor	<u>0.0</u>	<u>0.0</u>	<u>55.4</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>0.0</u>	<u>53.9</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>0.0</u>	<u>51.7</u>
23. Unit Forced Outage Rate	<u>100.0</u>	<u>100.0</u>	<u>36.3</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

8102180468

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289

UNIT TMI-1

DATE February 15, 1981

COMPLETED BY D. G. Mitchell

TELEPHONE (717) 948-8553

MONTH January 1981

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January 1981

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE February 15, 1981
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 TELEPHONE (717) 948-8553

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	810101	F	744	D	1				Regulatory Restraint Order

POOR ORIGINAL

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG
 0161)

⁵
 Exhibit I - Same Source

OPERATIONS SUMMARY

The Unit was in cold shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat System.

During the month a preplanned training/test program was conducted involving the primary plant systems and components. This program included heatup of the primary system to 185 F. During the period between 1/16/81, and 1/30/81, the following evolutions were conducted.

- January 16 - RCS filled (from partially drained)
 - 45 psig CRDM vent completed
- January 17 - Make-up system in operation (ran all make-up pumps)
 - RCS cleanup established (found acceptable crud levels)
 - Ran all RC pumps
 - Make-up system leak discovered (see LER-81-01 for detailed report)
- January 18 - Plant depressurized MU system secured

Between January 18, and 25, 1981, the make-up system leaks were repaired and the program continued as follows.

- January 25 - MU system in operation
 - Seal injection and cleanup chain established
 - RCS at 300 psig
- January 26 - Ran RC pumps
 - Vented CRDMs at 300 psig
- January 28 - Control rods withdrawn
 - Completed source range testing
- January 30 - Plant depressurized and nitrogen blanketing established on RCS

In addition to enabling hands-on training for most operators, several other benefits were gained from the program:

- 1) A program was established which verified no loose parts in the Reactor Coolant System.
- 2) The RCS was found to be leak tight at 300 psi, verifying the soundness of several primary system check valves on which maintenance has been performed (see previous monthly operating reports).
- 3) The ability to operate all CRDMs was verified (no binding was evident).
- 4) Subcritical multiplication was produced verifying the operability of the source range detectors and instrumentation.
- 5) Acceptable crud levels were found in the Reactor Coolant System and existing levels were reduced.

As expected a significant number of work items were generated as a result of the program. There were no major areas of concern uncovered however. It was concluded that, based on the training received and on the plant equipment performance, the program was very successful.

MAJOR SAFETY RELATED MAINTENANCE

During the month of January, 1981, the following major maintenance items were performed.

Spent Fuel System Loop "B" was in operation while weld repairs were performed on the "A" loop.

The following work was performed as part of the on-going intergranular stress corrosion cracking repair program:

- Weld #195 (Discharge of Spent Fuel Pump "A")
- Weld #503 (Suction of Spent Fuel Pump "A")
- Weld #501 (Suction of Spent Fuel Pump "A")
- Weld # 300 (Spent Fuel Cask Loading Pit drain/full line)

Repairs included either installation of a new spool piece or removing indication/rewelding. Functional test of the weld repairs is in progress.

Emergency Diesel Generator "B" annual inspection was performed with minor repairs performed. Balancing of the generator was performed to lower vibration readings.

Stop check valve work progressed with FW-V-12A/B being reassembled and valves torqued to final value. Various make-up system stop check valves, on which work was performed (see previous monthly operating reports), were retorqued.

Refueling Outage Surveillance Testing items were performed. Ten (10) snubbers were removed for functional testing and some were reinstalled after testing was performed satisfactorily. One snubber failed to lockup requiring an additional ten (10) more snubbers to be removed and tested. Work will continue into February.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).