NHC FURM JOD (7.77) LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 1(1) CONTROL BLOCK: 34111111 25 26 LICENSE TYPE J 0 0 - 0 0 0 0 - 0 0 0 4 CITMNS2 2 0 1 LICENSEE CODE LICENSE NUMBER CON'T 10 p p p 16 10 10 18 0 3 0 9 0 8 8 0 REPORT 1 2 81 (9) 600 10 0 1 SOURCE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) [While the 12U diesel was being run for surveillance testing, a 125 VDC (Bus 201A) ground] 0 2 alarm was received. The 13U diesel was verified operable and the plant was operated in 03 laccordance with the Action Statement 3.8.1.1.a of Technical Specifications. The ground 0 4 was corrected and the 12U diesel returned to operable status within 8 hours. 0 5 0 6 0 7 0 8 30 COMP CAUSE SUBCODE CODE CAUSE COMPONENT CODE SUBCODE CODE XIXIXIX 14 A (13) Ξ (16 Ε X L (12) 0 9 1.7 18 OCCURRENCE CODE REVISION SEQUENTIAL REPORT NO. YPE EVENT YEAR REPORT NO. LER/RO REPORT NUMBER 0 3 12 19 1810 0 32 28 COMPONENT MANUFACTURER SUBMITTED NPRD-4 PRIME COMP EFFECT METHOD TAKEN ACTION HOURS (22) FORM SUB SUPPLIER A 25 FI 01 11 N 24 0 (26 0 0 0 0 0 N 23 B (18) Ζ (21) (19 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27 The cause of the ground was frayed insulation on one wire in the Bendix connector which] 101 connects the speed sensor to the governor allowing a bare wire to contact the connector shell causing a short circuit. The connector leads were removed, cut back, stripped, reconnected and covered with heat shrink tubing 1 3 4 80 9 METHOD OF FACILITY (30) DISCOVERY DESCRIPTION (32 OTHER STATUS S POWER Surveillance Test B j(31 E (28) 1 0 0 29 NA 5 ACTIVITY CONTENT 80 LOCATION OF RELEASE (36) A YOUNT OF ACTIVITY (35 OF RELEASE NA 134 NA 6 (33) 80 35 4.4 11 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE 7 (38) 1010 NA 7 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 NA (40) 8 OSS OF OR DAMAGE TO FACILITY (43 80 TYPE DESCRIPTION NA Z (42) 9 PUBLICITY 8009160.64 NRC USE ONLY SUED A4 DESCRIPTION (45) NA 68 69 80. 10 J.S. Harris 203/447-1791 PHONE NAME OF PREPARED

BOSTON EDISON COMPANY PILGRIM NUCLEAR POWER STATION DOCKET NO. 50-293

Attachment to LER 80-049/03L-0

Description

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On August 10 and 11, 1980 SSW P208B tripped. On August 11 and 17, 1980 SSW P208E tripped. On each occasion the trips resulted from thermal overload of their respective breaker's B1533 and B1444. Immediate investigation revealed no cause for these trips. A detailed analysis was initiated to ascertain the cause of the trips and recommend corrective action.

Cause & Corrective Action

The analysis indicates that the major conditions contributing to these trips are:

- 1. An improvement in the SSW inlet water flow due to maintenance performed during the refuel IV outage.
- A reduction in heat transfer capability of the TBCCW system in non essential loops.
- 3. The number of SSW pumps in operation.
- 4. The inlet water level to the SSW pumps.
- 5. The inlet water temperature to the SSW pumps.

It is believed at this time that simultaneous operation of four SSW pumps with specific inlet water levels shifted the dynamic operating characteristic of B & E SSW pumps and resulted in protective trips of their respective breaker.

Permanent corrective action is being contemplated; meanwhile, inlet water temperature decrease has alleviated the problem.

Emergency mode of operation requires two pump operation and therefore the safety margin, in this mode has not been reduced.

Following the completion of the analysis and implementation of permanent corrective measures an update report will be issued.