

OPERATING DATA REPORT

DOCKET NO. 50-409  
 DATE 09-08-80  
 COMPLETED BY L.S. GOODMAN  
 TELEPHONE 608-689-2331

OPERATING STATUS

1. Unit Name: La Crosse Boiling Water Reactor
2. Reporting Period: 1000, 80-01-08 to 2400, 80-31-08
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 65.3
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level to Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744</u>	<u>855</u>	<u>94,970</u>
12. Number Of Hours Reactor Was Critical	<u>337.0</u>	<u>702.9</u>	<u>62,508.4</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>47</u>
14. Hours Generator On-Line	<u>264.8</u>	<u>4501.8</u>	<u>57,506.6</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>79</u>
16. Gross Thermal Energy Generated (MWH)	<u>32,466.7</u>	<u>597,266.4</u>	<u>7,382,046.9</u>
17. Gross Electrical Energy Generated (MWH)	<u>8,695</u>	<u>172,871</u>	<u>2,372,003</u>
18. Net Electrical Energy Generated (MWH)	<u>7,496</u>	<u>160,797</u>	<u>2,193,532</u>
19. Unit Service Factor	<u>35.6</u>	<u>76.9</u>	<u>61.6</u>
20. Unit Availability Factor	<u>35.6</u>	<u>76.9</u>	<u>60.6</u>
21. Unit Capacity Factor (Using MDC Net)	<u>21.0</u>	<u>57.2</u>	<u>48.1</u>
22. Unit Capacity Factor (Using DER Net)	<u>20.2</u>	<u>54.9</u>	<u>46.1</u>
23. Unit Forced Outage Rate	<u>64.4</u>	<u>14.2</u>	<u>7.1</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

ESTIMATED REFUELING OUTAGE, NOVEMBER 22, 1980 (6 Weeks)

25. If Shut Down At End Of Report Period, Estimated Date of Startup: JANUARY 3, 1981

26. Units In Test Status (Prior to Commercial Operation)	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-409

UNIT LACBWR

DATE 09-08-80

COMPLETED BY L. S. GOODMAN

TELEPHONE 608-689-2331

MONTH AUGUST 1980

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	37
2	37
3	37
4	37
5	37
6	37
7	37
8	13
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	12
20	17
21	19
22	12
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

POOR ORIGINAL

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August 1980

DOCKET NO. 50-409  
 UNIT NAME LACBWR  
 DATE 09-08-80  
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 TELEPHONE 608-689-2331

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
80-07	08-08-80	F	253.7	A	3	NA	CJ	INSTRU	FORCED CIRCULATION PUMPS TRIPPED DUE TO LOSS OF SEAL INJECTION FLOW. LOSS OF SEAL INJECTION FLOW WAS CAUSED BY LOW LEVEL IN RESERVOIR DUE TO LEVEL CONTROLLER MALFUNCTION. SEAL INJECT RESERVOIR LEVEL CONTROLLERS AND INDICATORS WERE RECALIBRATED AND THE SYSTEM TESTED WITH SATISFACTORY RESULTS. DURING THE SHUTDOWN A FLANGE LEAK ON CONTROL ROD DRIVE MECHANISM #28, THE 1A FCP SEAL, AND THE GLAND STEAM GENERATOR WERE ALSO REPAIRED.
80-08	08-22-80	F	225.5	A	3	NA	CH	INSTRU	THE SHUTDOWN RESULTED WHEN REACTOR FEED PUMP 1A TRIPPED DUE TO ELECTRICAL SHORT ON A PRINTED CIRCUIT CONTROL CARD CAUSED BY WATER ENTERING CONTROL ENCLOSURE. DECONTAMINATION OF OVER-

(Con'd on 2 of 2).....

1  
 F Forced  
 S Scheduled

2  
 Reason  
 A Equipment Failure (Explain)  
 B Maintenance or Test  
 C Refueling  
 D Regulatory Restriction  
 E Operator Training & License Examination  
 F Administrative  
 G Operational Error (Explain)  
 H Other (Explain)

3  
 Method  
 1 Manual  
 2 Manual Scram  
 3 Automatic Scram  
 4 Other (Explain)

4  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5  
 Exhibit F - Same Source

POOR ORIGINAL

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-409  
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REPORT MONTH AUGUST 1980

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Compage Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
									HEAD FLOOR CAUSED WATER TO DRIP THROUGH SMALL UNSEALED CONDUIT CHASE. THE CONDUIT CHASE WAS SEALED. WHILE THE PLANT WAS SHUT DOWN, MAINTENANCE WAS ALSO PERFORMED ON THE SEAL INJECTION SYSTEM AND THE MECHANICAL SEALS IN CONTROL ROD DRIVE MECHANISMS NOS. 5 AND 21.

<sup>1</sup> F - Forced  
 S - Scheduled

<sup>2</sup> Reason:  
 A - Equipment Failure (Explain)  
 B - Maintenance or Test  
 C - Refueling  
 D - Regulatory Restriction  
 E - Operator Training & License Examination  
 F - Administrative  
 G - Operational Error (Explain)  
 H - Other (Explain)

<sup>3</sup> Method:  
 1 - Manual  
 2 - Manual Scram  
 3 - Automatic Scram  
 4 - Other (Explain)

<sup>4</sup> Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup> Exhibit I - Same Source

## NARRATIVE SUMMARY OF OPERATING EXPERIENCE

AUGUST 1980

At the onset of the August 1980 reporting period, power generation was continuing at 85% Reactor Rated Thermal Power (37 MWe-Net). This operating level has been scheduled to extend core life to stretch optimum production until refueling becomes necessary.

On August 8, 1980, at 0843, the reactor automatically shut down on low power-to-flow when the Forced Circulation Pumps (FCP) tripped due to low seal inject flow. The reactor was returned to critical at 0436 on August 9, but was manually shut down at 1518 to repair a flange leak on Upper Control Rod Drive Mechanism No. 28. During the shutdown, corrective maintenance was also performed on the 1A FCP seal and the gland steam generator.

At 0940 on August 17, the reactor was returned to critical. At 2225 on August 18, the LACBWR turbine-generator was connected to the DPC grid and slow escalation commenced.

On August 22 at 1430, the reactor automatically shut down when the 1A Reactor Feed Pump (RFP) tripped due to water dripping onto and shorting out a portion of the RFP controls. The floor on the mezzanine level of the turbine building was being washed and water had dripped through a conduit chase, which has since been sealed. The plant had achieved 55% Reactor Rated Thermal Power (20 MWe-net) prior to the shutdown.

At 1712 on August 24, the reactor was returned to critical, but at 1449 on August 25 at .12% Reactor Rated Thermal Power, a scram occurred when the Forced Circulation Pumps tripped due to low seal injection leakoff flow, when attempting to shift Seal Injection Pumps, a necessary step in the plant startup. The low leakoff flow condition occurred because the Seal Injection Pumps' bypass valves did not compensate fast enough to maintain sufficient flow to the FCP's. The bypass valve controllers were checked thoroughly and tested with satisfactory results.

At 1750 on August 25, the reactor was returned to critical, but at 2056 it was manually shut down due to a mechanical seal leak on Upper Control Rod Drive #21. The plant remained in a shutdown condition throughout the duration of the August 1980 reporting period for seal repair work.

Significant maintenance items performed during the August 1980 reporting period are indicated on the attached Instrument and Electrical Maintenance and Mechanical Maintenance listings.

POOR ORIGINAL

MECHANICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LEAK OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
SHUTDOWN CONDENSER	FACILITY CHANGE NO. 22-80-1 (MR 3295)	LER 80-05	REMOVED VALVE AND PUMP; REPLACED WITH CAP.	UNTESTED FITTING	WELDED TEST CHAMBER ON 1/2" LINE
LOWER CRD IN POSITION #5 (FRAME #20)	PREVENTIVE (MR 3313)	OUTAGE 80-07	NA	NA	PERFORMED PREVENTIVE MAINTENANCE, TESTED AND STORED FOR SPARE
UPPER CRD IN POSITION #28	CORRECTIVE (MR 3306)	OUTAGE 80-07	FLANGE LEAK	EXCESSIVE LEAKAGE	PERFORMED CORRECTIVE AND PREVENTIVE MAINTENANCE AND REINSTALLED
1A FORCED CIRCULATION PUMP	CORRECTIVE (MR 3306)	OUTAGE 80-07	UPPER MECHANICAL SEALS WORN	EXCESSIVE LEAKAGE	REPLACED UPPER SEALS, FACE SEAL RING, SEAL SEAT RING, MECHANICAL SEAL AND O-RINGS
OFF-GAS AIR EJECTOR	FACILITY CHANGE NO. 55-80-2 (MR 3320)	OUTAGE 80-07	NA	NA	INSTALLED 2 1/2" GATE VALVE IN LOOP SEAL
1B SEAL INJECT SUCTION BLADDER	CORRECTIVE (MR 3316)	OUTAGE 80-07	VALVE STEM ON BLADDER LEAK	LOSE AIR FROM BLADDER	INSTALLED NEW BLADDER
GLAND STEAM GENERATOR	CORRECTIVE (MR 3192)	OUTAGE 80-07	GASKET-STEAM CUT	STEAM LEAK AT FLANGE	INSTALLED NEW GASKETS ON EACH SIDE OF HEAD
WELL NO. 4	CORRECTIVE (MR 3241)	NA	BROKEN DISCHARGE PIPE	LOSS OF WELL WATER	INSTALLED NEW 3" x 18" PIECE OF PIPE
UPPER CRD IN POSITION #21	CORRECTIVE (MR 3343)	OUTAGE 80-08	MECHANICAL SEAL FACE CUT	EXCESSIVE SEAL LEAKAGE	PERFORMED CORRECTIVE AND PREVENTIVE MAINTENANCE AND REINSTALLED
UPPER CRD IN POSITION #5	CORRECTIVE (MR 3348)	OUTAGE 80-08	MECHANICAL SEAL FACE CUT	EXCESSIVE SEAL LEAKAGE	PERFORMED CORRECTIVE AND PREVENTIVE MAINTENANCE AND REINSTALLED

## INSTRUMENT AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
SECURITY SYSTEM	CORRECTIVE MR3272, MR3289	NA	BURN-OUT	NO AUDIBLE INDICATION	REPLACED EXPLOSIVE DETECTOR SPEAKER
SECURITY SYSTEM	CORRECTIVE MR3294	NA	MISALIGNMENT	POOR QUALITY PICTURE	ADJUSTED CCTV CAMERAS #6 AND #8
SEAL INJECTION SYSTEM	CORRECTIVE MR3299	OUTAGE 80-08	TEST REQUEST	COMPLETED TESTS	COMPLETED OPERATIONAL TEST
TURBINE OIL PUMP BREAKER	CORRECTIVE MR3301	OUTAGE 80-07	BURN-OUT	BREAKER INOPERATIVE	REPLACED SPRING RELEASE COIL
WASTE WATER PUMP GAUGE	CORRECTIVE MR3211	OUTAGE 80-07	PLUGGED INLET	GAUGE INOPERATIVE	REPLACED GAUGE
FIRE DETECTION SYSTEM	PREVENTIVE	OUTAGE 80-07	TEST DUE	COMPLETED TEST	COMPLETED FIRE DETECTOR TEST
SAFETY SYSTEM	PREVENTIVE	OUTAGE 80-07	TEST DUE	COMPLETED TESTS	COMPLETED SAFETY SYSTEM TECH. SPEC. TESTS FOR CH 1, 2 AND WATER LEVEL #3
SEAL INJECTION RESERVOIR	CORRECTIVE MR3225	OUTAGE 80-07	TEST REQUESTED	COMPLETED TESTS	TESTED ALL CONTROL FUNCTIONS
CONDUCTIVITY RECORDER	CORRECTIVE MR3328	OUTAGE 80-07	NORMAL USAGE	INCORRECT PRINTOUT	CLEANED SWITCH AND SLIDE WIRE
SECURITY SYSTEM GATE	CORRECTIVE MR3323 MR3329	NA	BROKEN SWITCH	UNEXPECTED GATE CLOSURE	REPLACED CONTROL SWITCH
SECURITY SYSTEM CAMERA 1 AND 7	CORRECTIVE MR3339	NA	MISALIGNMENT	POOR PICTURE	REALIGNED CAMERS 1 & 7
FORCED CIRCULATION PUMP VIBRATION DEVICE	CORRECTIVE MR3332	OUTAGE 80-07	BROKEN CONNECTOR	INCORRECT READOUT	REPLACED DETECTOR
HYDROGEN ANALYZER	CORRECTIVE MR3334	NA	CONTAMINATION	IMPROPER READING	CLEANED AND CALIBRATED SYSTEM
MAIN EXCITER AUTO VOLTAGE CONTROL	CORRECTIVE MR3333	OUTAGE 80-07	BURNOUT	NO AUTO EXCITATION CONTROL	REPLACED BOOST AMPLIFIER

## INSTRUMENT AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULT	
DIESEL FIRE PUMP	CORRECTIVE MR3330	LER 80-08	LOW CELL ON BATTERY	REPLACEMENT DUE	REPLACED START BATTERIES
NUCLEAR INSTRUMENTATION	PREVENTIVE	NA	TEST DUE	COMPLETED TESTS	COMPLETED N-5 THROUGH N-9 PLUS HOT SHORT
CONTROL ROD DRIVE 28	CORRECTIVE MR3341	OUTAGE 80-08	UNKNOWN	NO GAS PRESSURE READING	REPLACED PRESSURE TRANSMITTER
SECURITY SYSTEM ZONE 10	CORRECTIVE MR3326	NA	MISALIGNMENT	NO ALARM	ADJUSTED MAGNETIC SWITCH
SECURITY SYSTEM STACK SHACK CARD READER	CORRECTIVE MR3338	NA	UNKNOWN	NO ENTRY	ADJUSTED CARD READER
FEED PUMP CONTROL	CORRECTIVE MR3342	OUTAGE 80-08	CONTAMINATION & ELECTRICAL SHORT	AMPLIFIER BURN-OUT	REPAIRED CONTROL AMPLIFIERS
CONTROL ROD DRIVE #28	CORRECTIVE MR3313	OUTAGE 80-07	WATER IN TRANSMITTER	DEFECTIVE TRANSMITTER	REPLACED TRANSMITTER
CONTROL ROD DRIVE SELECTOR SWITCH	CORRECTIVE MR3335	OUTAGE 80-08	DEFECTIVE SWITCH	HARD OPERATION	REPLACED SWITCH WITH NEW UNIT
NUCLEAR INSTRUMENT CH. N-1	CORRECTIVE MR3347	OUTAGE 80-08	INOPERATIVE	INCORRECT READING ON METER	REPLACED TUBES AND INSTALLED SPARE
FORCED CIRCULATION PUMP TEMPERATURE READOUT 1B	CORRECTIVE MR3343	OUTAGE 80-08	INOPERATIVE	NO READING ON METER	REPAIRED OPEN WIRES ON THERMOCOUPLE
TURBINE ROTOR JACKING GEAR INTERLOCK	CORRECTIVE MR3336	OUTAGE 80-08	WEAR	SWITCH INOPERATIVE	REPLACED PRESSURE SWITCH
UNDERWATER LAMPS	CORRECTIVE MR3355	OUTAGE 80-08	OUTAGE	BURNED OUT LAMPS	REPLACED LAMPS



REFUELING INFORMATION REQUEST

1. Name of Facility:

La Crosse Boiling Water Reactor (LACBWR)

2. Scheduled Date for Next Refueling Shutdown:

The tentatively scheduled date for the next refueling shutdown (EOC-VI) is November 22, 1980.

3. Scheduled Date for Restart Following Refueling:

The tentatively scheduled date for subsequent reactor startup is January 3, 1981.

4. Will Refueling or Resumption of Operation Thereafter Require a Technical Specification Change or Other License Amendment?

If answer is yes, what in general, will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

No Technical Specification Change or license amendment will be needed for refueling or subsequent resumption of operation. The Safety Review Committee review of the reload fuel design and core configuration will be conducted prior to reload.

5. Scheduled Date(s) for Submitting Proposed Licensing Action and Supporting Information.

NA

6. Important Licensing Considerations Associated with Refueling, e.g. New or Different Fuel Design or Supplies, Unreviewed Design or Performance Analysis Methods, Significant Changes in Fuel Design, New Operating Procedures.

None

7. The Number of Fuel Assemblies (a) in the Core and (b) in the Spent Fuel Pool.

Core Loading: 72 Fuel Assemblies

Spent Fuel Storage Pool Loading: 133 Irradiated Fuel Assemblies

REFUELING INFORMATION REQUEST - (Cont'd)

8. The Present Licensed Spent Fuel Pool Storage Capacity and the Size of any Increase in Licensed Storage Capacity that has been Requested or so Planned, in Number of Fuel Assemblies.

LACBWR is presently in the process of increasing the spent fuel pool storage capacity from 134 to 440 fuel assemblies.

9. The Projected Date of the Last Refueling that can be Discharged to the Spent Fuel Pool Assuming the Present Licensed Capacity.

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