

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | W | I | P | B | H | L | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | _____ | 5
7 8 9 14 15 25 26 30 57 58
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

CON'T
01 | R | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 6 | 6 | 7 | 1 | 1 | 2 | 1 | 0 | 8 | 0 | 8 | 0 | 1 | 0 | 9 | 8 | 1 | 1 | 9
7 8 60 61 68 69 74 75 80
REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
02 | During refueling outage surveillance testing of safety-related shock
03 | suppressors of Unit 1, according to Technical Specification 15.4.13.2
04 | on 12/10/80, snubber 1-HS-14 (utility identification), located on the
05 | power-operated relief valve header, was determined to be inoperable.
06 | This event is reportable according to Technical Specification
07 | 15.6.9.2.A.9 and similar to LER 78-015/01T-0, 77-011/01T-0.
08 | _____
7 8 9 80

09 | S | H | 11 | E | 12 | F | 13 | S | U | P | O | R | T | 14 | D | 15 | Z | 16
7 8 9 10 11 12 13 18 19 20
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE
17 | 8 | 0 | 21 | 22 | 23 | 0 | 1 | 5 | 24 | 26 | 27 | 0 | 1 | 28 | 29 | T | 30 | 31 | 1 | 32
7 8 9 21 22 23 24 26 27 28 29 30 31 32
LER NO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.
18 | A | 18 | Z | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | 22 | Y | 23 | Y | 24 | N | 25 | G | 2 | 5 | 5 | 26
7 8 9 33 34 35 36 37 40 41 42 43 44 47
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | The Grinnel I.D. No. 3146, Figure 200/11K, 2-1/2" x 5" (bore and stroke)
11 | snubber was determined to be inoperable due to a low level in its reser-
12 | voir, resulting from piston rod O-ring seal leakage. The snubber was
13 | repaired on 12/10/80, and tested and returned to service on 12/11/80.
14 | Similar snubbers were checked in accordance with TS 15.4.13.2 on 12/11/80.
7 8 9 80

15 | H | 28 | 0 | 0 | 0 | 29 | NA | 30 | B | 31 | Surveillance test | 32
7 8 9 10 12 13 44 45 46 80
FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION
16 | Z | 33 | Z | 34 | NA | 35 | NA | 36
7 8 9 10 11 44 45 80
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE
17 | 0 | 0 | 0 | 37 | Z | 38 | NA | 39
7 8 9 11 12 13 80
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION
18 | 0 | 0 | 0 | 40 | NA | 41
7 8 9 11 12 80
PERSONNEL INJURIES NUMBER DESCRIPTION
19 | Z | 42 | NA | 43
7 8 9 10 80
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION
20 | N | 44 | NA | 45
7 8 9 10 80
PUBLICITY ISSUED DESCRIPTION
NRC USE ONLY
80

8101270710
NAME OF PREPARER C. W. Fay PHONE 414/277-2811
7 8 9 68 69 80 81

ATTACHMENT TO LICENSEE EVENT REPORT NO. 80-015/01T-0

Wisconsin Electric Power Company
Point Beach Nuclear Plant Unit 1
Docket No. 50-266

During refueling outage surveillance testing of safety-related shock suppressors of Unit 1, in accordance with Technical Specification 15.4.13.2 on December 10, 1980, snubber 1-HS-14 (utility identification), located on the power-operated relief valve header, was found with a low fluid level in its reservoir. The Grinnel I.D. No. 3146, Figure 200/11k, 2-1/2" x 5" (bore and stroke) snubber was determined to be inoperable due to leakage past the piston rod O-ring seals.

Snubber 1-HS-14 was repaired on December 10, 1980 and retested satisfactorily on December 11, 1980. In accordance with Technical Specification 15.4.13.2, an additional 10% of all snubbers of the same type as 1-HS-14 (one snubber) were inspected. Snubber 1-HS-14 was inspected with satisfactory results on December 11, 1980.

This event is reportable in accordance with Technical Specification 15.6.9.2.A.9 since remedial action was required to prevent operation in a less conservative manner than that assumed in the safety analysis report.

The initial notification of this event was made to the Duty Officer in the Operations Center in Bethesda on December 26, 1980 when, during a review of refueling maintenance records, it was realized that this event was reportable in accordance with the Technical Specification listed. The followup Licensee Event Report was incorrectly dated December 23, 1980. The corrected initial Licensee Event Report date should be December 29, 1980.