NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: ☐☐☐☐☐☐ ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 LICENSEE CODE: P A B V S 1 ② LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 ③ LICENSE TYPE: 4 1 1 1 1 ④ CAT: ⑤

CON'T 01 REPORT SOURCE: L ⑥ DOCKET NUMBER: 0 5 0 0 0 3 3 4 ⑦ EVENT DATE: 0 1 0 7 8 1 ⑧ REPORT DATE: 0 1 2 1 8 1 ⑨

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

02 With the reactor shutdown, an operator was performing a manipulation of the seal
03 leak-off valves at 1105 hours when the seal flow dropped below the minimum on the
04 1C Reactor Coolant Pump, the only RCP running. The pump was manually tripped, as
05 required. However, Technical Specification 3.4.1.1 requires at least one pump
06 running. The pump was restarted within 20 minutes. There was a minimum of decay
07 heat in the core and subcooled conditions were maintained. The health and safety
08 of the public and site personnel was not jeopardized.

09 SYSTEM CODE: C B ⑪ CAUSE CODE: B ⑫ CAUSE SUBCODE: A ⑬ COMPONENT CODE: Z Z Z Z Z Z ⑭ COMP. SUBCODE: Z ⑮ VALVE SUBCODE: Z ⑯

⑰ LER/RO REPORT NUMBER: EVENT YEAR: 8 1 — SEQUENTIAL REPORT NO.: 0 0 2 / OCCURRENCE CODE: 0 1 REPORT TYPE: T — REVISION NO.: 0

ACTION TAKEN: E ⑱ FUTURE ACTION: F ⑲ EFFECT ON PLANT: Z ⑳ SHUTDOWN METHOD: Z ㉑ HOURS ㉒: 0 0 0 0 ATTACHMENT SUBMITTED: Y ㉓ NPRD-4 FORM SUB.: N ㉔ PRIME COMP. SUPPLIER: Z ㉕ COMPONENT MANUFACTURER: Z 9 9 9 ㉖

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

10 The cause of the unstable seal flow occurred during the valving attempt to transfer
11 the RCP seal leak-off flow path. The valving involved is sensitive enough to cause
12 this instability. The plant engineering group is investigating a proposed
13 Westinghouse solution to this problem.
14

15 FACILITY STATUS: C ㉘ % POWER: 0 0 0 ㉙ OTHER STATUS ㉚: N/A METHOD OF DISCOVERY: A ㉛ DISCOVERY DESCRIPTION ㉜: Operator observation

16 ACTIVITY RELEASED: Z ㉝ CONTENT OF RELEASE: Z ㉞ AMOUNT OF ACTIVITY ㉟: N/A LOCATION OF RELEASE ㊱: N/A

17 PERSONNEL EXPOSURES NUMBER: 0 0 0 ㊲ TYPE: Z ㊳ DESCRIPTION ㊴: N/A

18 PERSONNEL INJURIES NUMBER: 0 0 0 ㊵ DESCRIPTION ㊶: N/A

19 LOSS OF OR DAMAGE TO FACILITY ㊸ TYPE: Z ㊷ DESCRIPTION: N/A

20 PUBLICITY ISSUED: N ㊹ DESCRIPTION ㊺: N/A

NRC USE ONLY ☐☐☐☐☐☐☐☐☐☐☐☐

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8101270593

Attachment To LER 81-002/01T
Beaver Valley Power Station
Duquesne Light Company
Docket No. 50-334

Because of a change made to the Reactor Coolant System (RCS) pressure requirements due to the overpressurization protection system, seal leak-off from the Reactor Coolant Pumps (RCPs) must be discharged to the Primary Drains Tank to provide less back-pressure on the No. 1 RCP seal than the Volume Control Tank (VCT). When it becomes necessary to return the seal leak-off flow to the normal system arrangement, the flow is directed to the VCT. The valving involved in this transfer can cause the flow to become unstable. If the flow is less than .2 gpm during the transfer, the operator must trip the RCP to protect the pump seal. Once the RCS pressure is increased to normal system pressure, the seal leak-off flow is stable and no further problems are encountered.

Westinghouse has proposed a partial solution to this problem. They have suggested a finer flow measurement system which would help alleviate the control problem. This finer indication, along with throttle valves that are more suitable for very low flows, will eliminate valving and control problems. The Onsite Engineering Group is evaluating this problem.