LICENSEE EVENT REPORT

EIGENGEE ET EIT HEIGHT
CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1 N C B E P 1 2 0 0 - 0 0 0 0 - 0 0 3 4 1 1 1 1 1 6 57 CAT 58 5
CON'T 0 1 6 0 5 0 - 0 3 2 5 7 0 1 0 3 8 1 8 0 1 1 4 8 1 9 7
During a normal reactor startup and heatup, safety-relief valve B21-F013G, Model 67F.
0 3 lifted at 717 psig and reseated three minutes later at 465 psig. Right after seating,
[0]4] [the valve again opened but immediately reclosed. Due to this event, the resulting
[0]5 swell created by the pressure reduction, caused the feed pump to trip on high vessel
[0]6 [level (+58 inches), which resulted in a reactor scram on low level (+12.5 inches).
07
Technical Specifications 3.4.2, 6.9.1.8i
SYSTEM CODE CODE SUBCODE SUBCO
TO LER RO EVENT YEAR REPORT NO. CODE TYPE NO.
ACTION FUTURE EFFECT SHUTDOWN HOURS (22) ATTACHMENT FORM SUB. PRIME COMP. COMPONENT MANUFACTURER C 18 C 19 A 20 C 21 0 0 7 1 N 23 Y 24 N 25 T 0 2 0 26 33 34 34 35 36 37 37 37 40 41 23 42 43 43 45 47
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The pilot valve was sent to Wyle Laboratory to determine its cause for failure. The
valve was found to have high second stage and pilot leakage. The pilot valve was
replaced with one from stock. The pilots valves are scheduled to be replaced with a
new design pilot valve during the next scheduled outage of sufficient length on each
114 junit.
7 8 9 FACILITY STATUS 1 5 C 28 0 0 5 29 NA A A 31 Operator Surveillance 80 NA A 45 46
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 NA
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) NA
7 8 9 PERSONNEL INJURIES 80
NA PUR URGNAL SO
LOSS OF OR DAMAGE TO FACILITY 43 TYPE DESCRIPTION NA NA
PUBLICITY ISSUED DESCRIPTION (45) NA NRC USE ONLY NA
7 8 9 10 68 69 80.5 P M Poully Ir 919-457-9521 9
8101220402 PHONE R. M. FOUIK, ST.