



Public Service Company of Colorado

16805 ROAD 19½
PLATTEVILLE, COLORADO 80651

January 14, 1981
Fort St. Vrain
Unit No. 1
P-81014

Mr. Karl V. Seyfrit, Director
Nuclear Regulatory Commission
Region IV
Office of Inspection and Enforcement
611 Ryan Plaza Drive
Suite 1000
Arlington, Texas 76012

Reference: Facility Operating License
No. DPR-34

Docket No. 50-267

Dear Mr. Seyfrit:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/80-075, Final, submitted per the requirements of Technical Specification AC 7.5.2(b)2.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/80-075.

Very truly yours,

Don Warembourg
Don Warembourg
Manager, Nuclear Production

DW/clb

Enclosure

cc: Director, MIPC

8101210 406

5

A002
S 1/1

REPORT DATE: January 14, 1981

REPORTABLE OCCURRENCE 80-075

OCCURRENCE DATE: December 15, 1980

ISSUE 0

Page 1 of 7

FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
16805 WELD COUNTY ROAD 19 1/2
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/80-075/03-L-0

Final

IDENTIFICATION OF
OCCURRENCE:

During the period from December 15, 1980, through December 31, 1980, the plant was operated with total primary coolant oxidants (the sum of carbon monoxide, carbon dioxide, and water) greater than 10 parts per million and average core outlet temperature greater than 1200°F on seven separate occasions.

These events constitute operation in a degraded mode of LCO 4.2.10 and are reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)2.

EVENT
DESCRIPTION:

From December 15, 1980, through December 31, 1980, the plant was operated in a degraded mode of LCO 4.2.10 on seven separate occasions as described below:

1. (Refer to Figure 1.) On December 14, the plant power level was being varied as part of a test (RT-500H). Upon completion of the test, power level was increased on the 15th and primary coolant oxidants also increased. The reason was not clear, and to verify that no steam generator tube leakage was present, the plant power level was reduced and the steam generators were sequentially dumped and checked. This was conveyed to the Nuclear Regulatory Commission via a Significant Event Notification. No leakage was found, and the plant was returned to power.
2. (See Figure 2.) This incident is a continuation of incident #1. The return to power caused the oxidants to increase and resulted in total oxidants greater than 10 ppm.
3. (See Figure 3.) Following the decrease of oxidants after event #2, the plant continued at power levels of about 68% until December 24, 1980, when oxidants again increased. The oxidants gradually decreased again until 2200 hours on December 25, when the total oxidants were 9.9 ppm.

EVENT

DESCRIPTION: (Cont'd)

4. (See Figure 4.) On December 24, at midnight, the total oxidants again exceeded 10 ppm for one reading. Shortly thereafter, the plant was taken off line to repair a leak at a steam generator feedwater trim valve.
5. (See Figure 4.) On the return to power following the trim valve repair, the total oxidants again followed power and exceeded 10 ppm. While the oxidants were still increasing, the turbine and Loop 2 tripped due to a control problem with Loop 2 helium circulator steam bypass valve, PV-2244.
6. The repair to PV-2244 was completed and the plant returned to power.
7. The oxidants again followed the power increase and exceeded 10 ppm.
7. This continued until December 31, 1980, when the power was reduced to repair a valve.

CAUSE

DESCRIPTION:

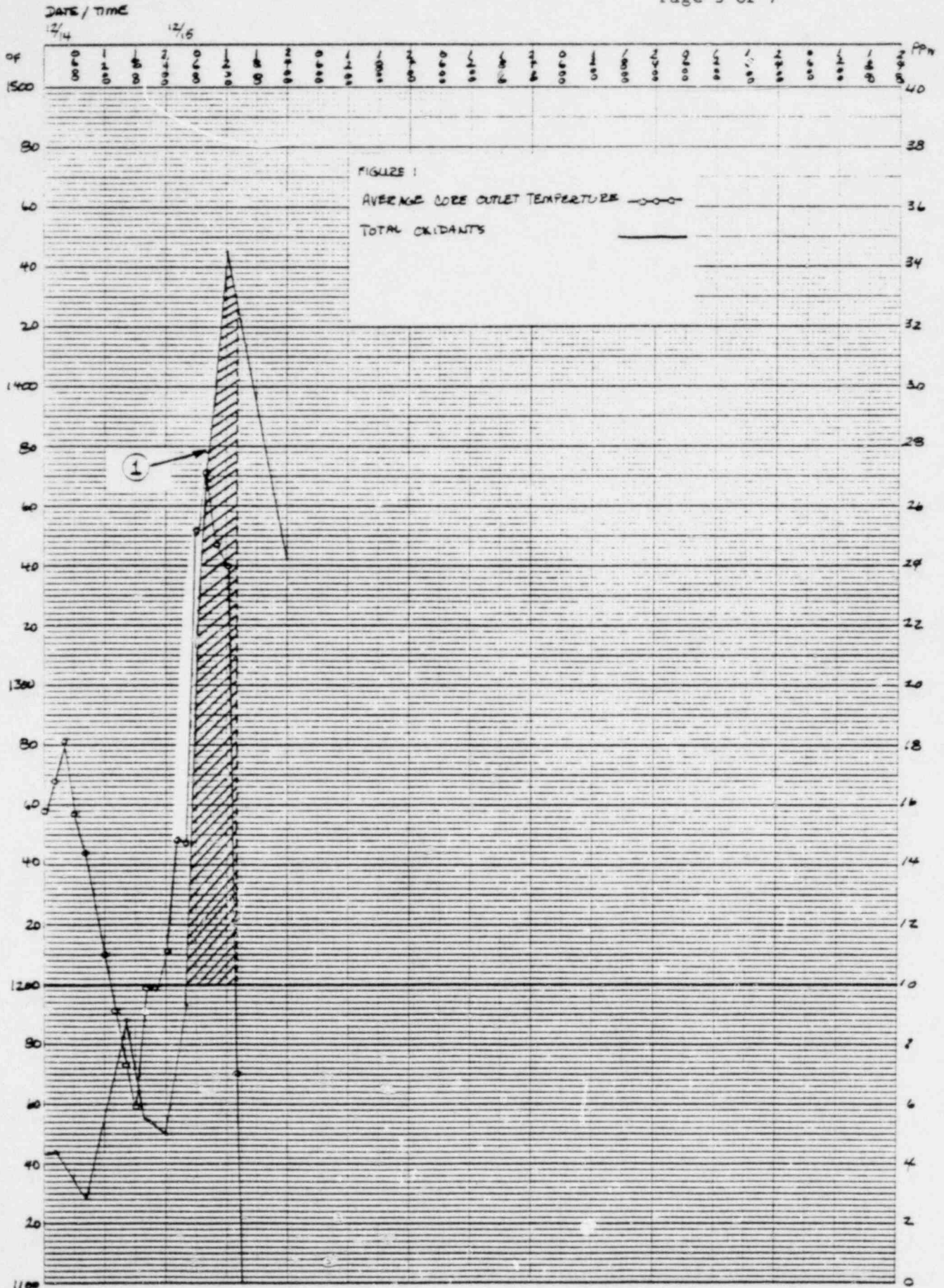
The primary coolant oxidants experienced during these seven periods is being attributed to high moisture in the helium storage tanks. This is introduced into the primary coolant as the stored helium is used as makeup helium.

CORRECTIVE

ACTION:

The original corrective action was to take the plant off line and sequentially dump the steam generators to verify there was no steam generator tube leak. When this was established, sources of makeup helium were investigated and high moisture found in the storage tank. The storage tank was purged and dry helium obtained.

No further corrective action is anticipated or required.



POOR ORIGINAL

12/83

2/9

2/20

 $\frac{2}{3}$

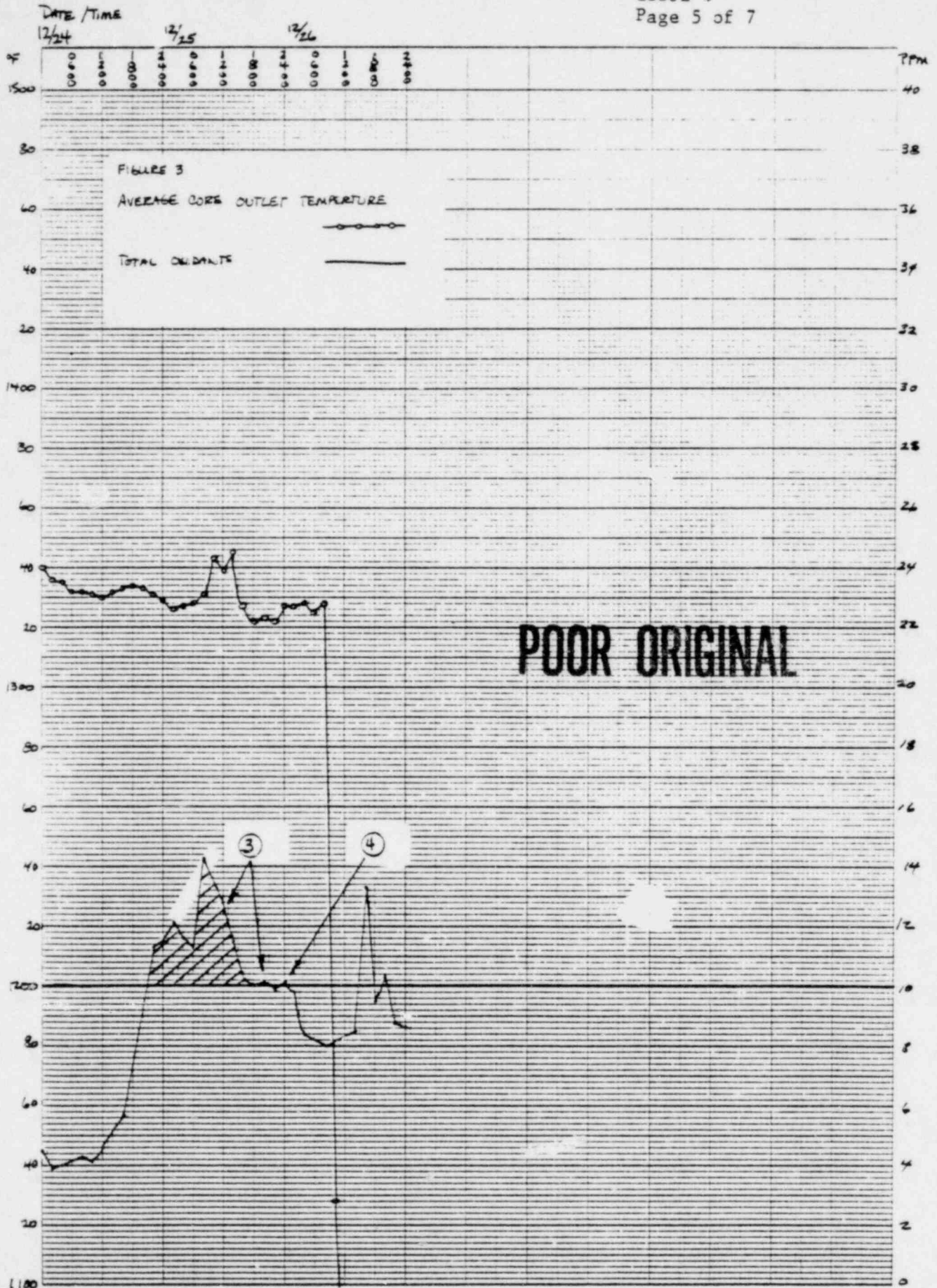
12/12

FIGURE 2

AVERAGE CORE OUTLET TEMPERATURE

TOTAL OXIDANTS

POOR ORIGINAL

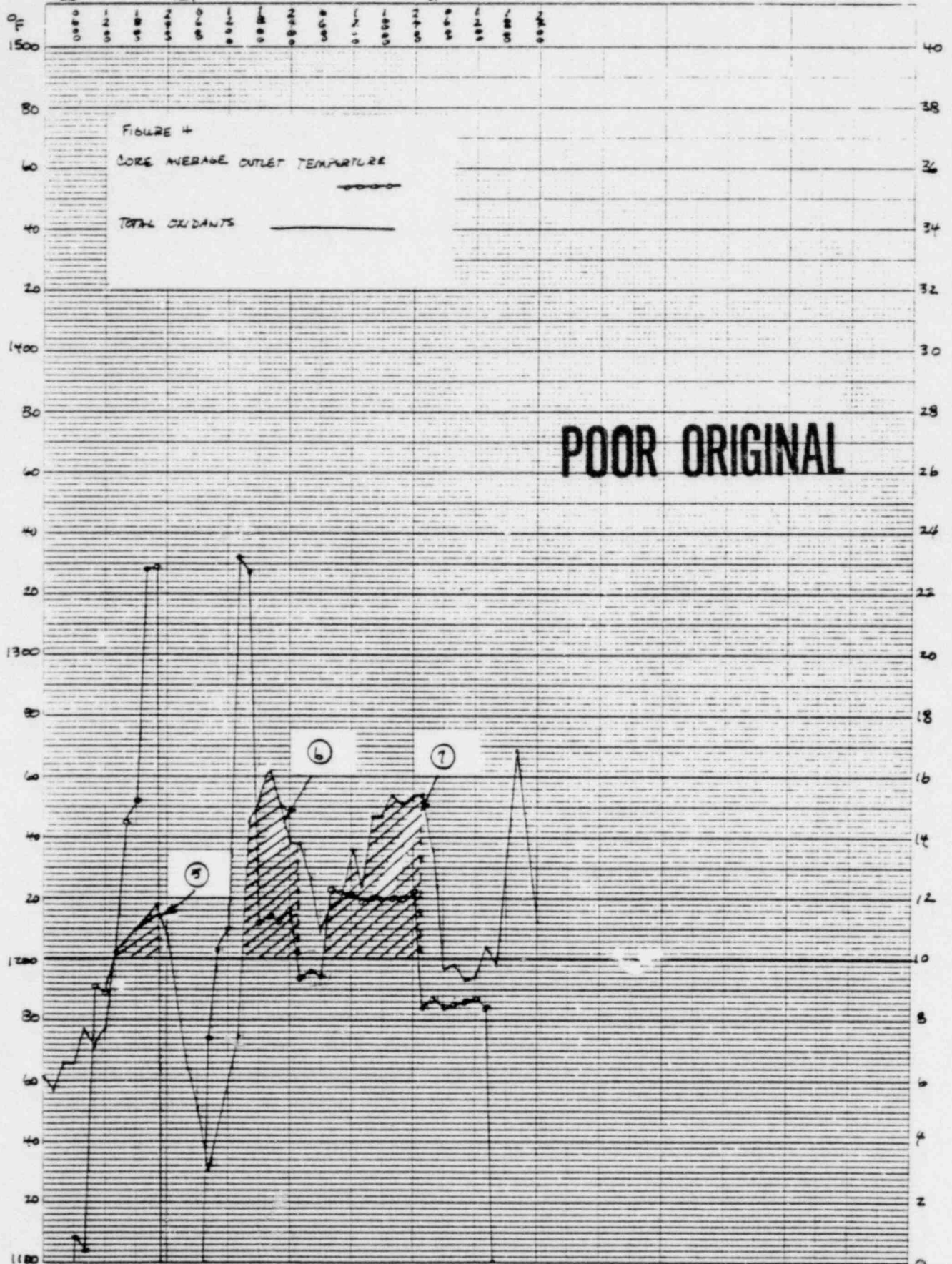


12/28

12/39

2/30

12/31



PREPARED BY: Asa B. Reed
Asa B. Reed
Technical Services Technician

REVIEWED BY: J. W. Galm
J. W. Galm
Technical Services Supervisor

REVIEWED BY: Frank M. Mathie
Frank M. Mathie
Operations Manager

APPROVED BY: Don Warembourg
Don Warembourg
Manager, Nuclear Production