UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-~~399~~ 339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

License No. NPF-7
Amendment No. 2

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric and Power Company (VEPCO), dated November 20, 1980, and supplemented by letters, dated December 15 and 18, 1980, for the North Anna Power Station, Unit No. 2 license complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the license, as amended, the provisions of the Act and the regulations of the Commission,

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changing the following conditions contained in paragraph 2.C of Facility Operating License NPF-7 to read:

   (12) VEPCO shall implement the following modifications related to IE Bulletin 79-27 "Loss of Non-Class IE Instrumentation and Control Power System Bus During Operation" as specified in VEPCO's letters, dated May 30 and July 9, 1980 on the following schedule:

(a) Prior to startup following the November 1, 1980 outage for Fire Protection Modifications:

Item 2 - Alternate Feed for Annunciators

Items 3, 4, 7 and 8: Alternate Power Supply for Vital SOV and Vital Instrument Panels.

Items 9 and 10: Loss of Voltage for Semi-Vital Buses

(b) Within six months from date of issuance of this license:

Item 6: Alternate Power to Gaitronics

Items 11, 12, 13 and 14: Change to Voltage Indication - 125 VDC Buses

(c) Prior to startup following the first refueling outage:

Items 1 and 5: Diverse Power Supply for T_h and T_c

(20) TMI Action Plan Conditions

Each of the following conditions references the appropriate section of Supplement No. 11 to the Safety Evaluation Report (NUREG-0053) for the North Anna Power Station, Unit 2, dated August 1980.

(a) Control Room Design Review (Section 22.2 Item I.D.1)

Within the schedule requirements of NUREG-0737, VEPCO shall submit an evaluation of the benefits of installing data recording and logging equipment in the control room to correct deficiencies associated with the trending of important parameters on strip chart recorders in use at most nuclear power plants, as part of their one-year control room design review.

(21) TMI Action Plan Dated Conditions

Each of the following conditions references the appropriate section of Supplement No. 11 to the Safety Evaluation Report (NUREG-0053) for the North Anna Power Station, dated August 1980, and shall be completed to the satisfaction of the NRC.

(c) Reactor Coolant System Vents (Section 22.2 Item II.B.1)

VEPCO shall submit procedural guidelines and analytical bases for the reactor coolant system vents. The reactor coolant system vents shall be installed no later than the implementation schedule of NUREG-0737.

(d) Plant Shielding (Section 22.3 Item II.B.2)

VEPCO shall complete modifications to assure adequate access to vital areas and protection of safety equipment following an accident resulting in a degraded core no later than the implementation schedule of NUREG-0737.

(e) Post-Accident Sampling (Section 22.3 Item II.B.3)

VEPCO shall complete corrective actions needed to provide the capability to promptly obtain and perform radioisotopic and chemical analysis of reactor coolant and containment atmosphere samples under degraded core conditions without excessive exposure no later than the implementation schedule of NUREG-0737.

(f) Relief and Safety Valve Test Requirements (Section 22.3 Item II.D.1)

VEPCO shall complete tests to qualify the reactor coolant system relief and safety valves under expected operating conditions for design basis transients and accidents no later than the implementation schedule of NUREG-0737.

(h) Containment Dedicated Penetrations (Section 22.3 Item II.E.4.1)

VEPCO shall install redundant remote actuated valves in series to isolate the containment vacuum pumps from the combustible gas control system. VEPCO shall also convert the manual valves in the hydrogen recombiner piping to remote manual actuation no later than the implementation schedule of NUREG-0737.

(i) Additional Accident Monitoring Instrumentation (Section 22.3 Item II.F.1)

VEPCO shall install and demonstrate the operability of instruments for continuous indication in the control room of the following variables. Each item shall be completed by the specified date in the condition:

(i) Containment pressure from 0 psia to three times the design pressure of the containment no later than the implementation schedule of NUREG-0737;

(ii) Containment water level from (1) the bottom to the top of the containment sump, and (2) the bottom of the containment to a level equivalent to 600,000 gallons of water no later than the implementation schedule of NUREG-0737.

(iii) Containment atmosphere hydrogen concentration from 0 to 10 volume percent shall be installed no later than the implementation schedule of NUREG-0737; and the hydrogen sampling system to be used in the interim will be installed no later than January 1, 1981;

(iv) Containment radiation up to 10^7 R/hr. no later than the implementation schedule of NUREG-0737; and

(v) Noble gas effluent from each potential release point from normal concentrations to 10^5 µCi/cc (Xe-133) no later than the implementation schedule of NUREG-0737.

VEPCO shall also provide capability for continuous sampling and for onsite analysis of the radioiodine and particulate effluent samples no later than the implementation schedule of NUREG-0737.

Until the above installation is completed, VEPCO shall use interim monitoring procedures and equipment.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Darrell G. Eisenhut, Director
Division of Licensing

Date of Issuance:
December 29, 1980