UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report No. 50-369/80-36

Licensee: Duke Power Company
422 South Church Street
Charlotte, NC 28242

Facility Name: McGuire Nuclear Station, Unit 1

Docket No. 50-369

License No. CPPR-83

Inspection at McGuire site near Charlotte, North Carolina

Inspector: A R Herdt 11/24/80
K. N. Economos Date Signed

Approved by: A R Herdt 11/24/80
A. R. Herdt, Section Chief, RC&ES Branch Date Signed

SUMMARY

Inspection on November 3-6, 1980

Areas Inspected

This routine, announced inspection involved 25 inspector-hours on site and corporate office in the areas of preservice inspection data review and, evaluation.

Results

No violations or deviations were identified in the areas inspected.

DETAILS

1. Persons Contacted

Licensee Employees

*J. O. Barbour, Quality Assurance Manager Operations
*C. B. Cheezem, Inservice Inspection Engineer

Other licensee employees contacted included technicians, and office personnel.

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on November 6, 1980 with those persons indicated in Paragraph 1 above.

3. Licensee Action on Previous Inspection Findings

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Preservice Inspection Data Review and Evaluation

The preservice (baseline) inspection (PSI) of class 1 and 2 portions of the Nuclear Steam Supply System of McGuire Unit 1 was performed by B&W customer Service Department. The PSI program including applicable procedures were written to comply with the requirements of ASME Section XI (71W72). The licensee has submitted approximately thrity-one (31) requests for relief involving welds and/or components where accessability and other reasons has made it impractical to conduct the code required preservice inspection. Relief on these items was granted by letter on July 11, 1980 (S. S. Pawlicki to W. J. Youngblood, Division of Licensing).

The inspector reviewed results of the PSI, documented in summary form and, in supportive field data sheets. Specific areas of review were as follows:

1. Reactor Pressure vessel
2. Pressurizer
3. Steam generators
4. Reactor coolant pumps
5. Class 1 Piping and supports, Reactor coolant system - hot and cold legs, crossovers.
6. Class 1 valves

7. Class 2 piping

In order to confirm results of certain UT inspection results, the inspector selected the following welds for a review of construction radiographs.

Weld No.	System
1 MI-486-2	Safety Injection
NI1F-683	Safety Injection
NI1F-685	Safety Injection
1NI-483-1	Safety Injection
1NI-467-2	Safety Injection
NC1F-108	Safety Injection
NC1F-1072	Reactor Coolant
NC1F-1072	Reactor Coolant
NC1F-3613-3092	Reactor Coolant
NC1F-2643	Reactor Coolant
1ND-193-2	Residual heat removal
CF1F-712	Feedwater
CF1F-474	Feedwater
CF1F-467	Feedwater
CF1F-461	Feedwater
CF1F-45	Feedwater
CF1F-106-VLV29	Feedwater
MK2 to MK3 (C F 94, A)	Feedwater
MK 4 to MK 2 (C F 94, B)	Feedwater
MK 1 to MK 4 (C F 94, C)	Feedwater

Within these areas the following items were discussed with the licensee's representative who agreed to persue them further.

1. Visual examination of the RPV closure head washers disclosed that washer #42 exhibited porosity.

2. An arc. strike detected in weld number NI1F2008, of the safety injection system, identified as PSI figure 4.4 66.23.

3. Stud #3 on valve 1NI 38, FL-10 identified as PSI figure 4.3.25. did not have proper thread engagement.

4. Class 1 Valves

   a. The #6 nut on the apposite flange of valve 1NC-2 was not properly torqued.

   b. On the outlet side of valve 1NC-1, nuts (#1, #8, #9, #10 and #11) were not sufficiently engaged on their respective studs.

c. Valve bolts for figure numbers 6.5.57, 6.5.58 and 6.5.59 were not inspected (visual).

5. Internal valve Body Examination

The code required visual examination was substituted for code and DPC required documentation. However the request for relief, referenced MC-1-106 in the PSI summary report, applies to hydrostatic testing.

6. Containment Spray Heat Exchange Nozzle Welds

MT was substituted for the code required UT examinations. However the request for relief MC-1-025, referenced in the PSI summary report, applies only to RHR heat exchangers National Board #635 and 636.

7. Containment Spray Pump 1A Studs

Visual examination of 1A pump studs #1-#24 disclosed that they were not fully engage in their respective nuts.

The inspector stated that this matter would be identified as inspector followup item 369/80-36-01 "Disposition of PSI Related Findings".