



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

November 10, 1980

Those on Attached Address Listing:

This IE Information Notice No. 80-41 is provided as an early notification of a possibly significant matter. It is expected that recipients will review the information for possible applicability to their facilities. No specific action or response is requested at this time. If further NRC evaluations so indicate, an IE Circular or Bulletin will be issued to recommend or request specific licensee actions. If you have questions regarding this matter, please contact the this office.

Sincerely,

Gen W. Roy
for James G. Keppler
Director

Enclosure: IE Information
Notice No. 80-41

cc w/encl:
Mr. D. W. Kane,
Sargent & Lundy
Mr. Gary N. Wright, Chief
Division of Nuclear Safety
Reproduction Unit NRC 20b

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RIII's CORPORATE ADDRESSES FOR OPERATING LICENSEES AND CONSTRUCTION PERMIT HOLDERS
(BWR's and PWR's)

Docket No. 50-329
Docket No. 50-330

Consumers Power Company
ATTN: Mr. James W. Cook
Vice President
Midland Project
1945 West Parnall Road
Jackson, MI 49201

cc w/encl:
Central Files
Director, NRR/DPM
Director, NRR/DOR
PDR
Local PDR
NSIC
TIC
Ronald Callen, Michigan
Public Service Commission
Myron M. Cherry, Chicago

Docket No. 50-409

Dairyland Power Cooperative
ATTN: Mr. F. W. Linder
General Manager
2615 East Avenue - South
La Crosse, WI 54601

cc w/encl:
Mr. R. E. Shimshak,
Plant Superintendent
Central Files
Director, NRR/DPM
Director, NRR/DOR
AEOD
Resident Inspector, RIII
PDR
Local PDR
NSIC
TIC
Mr. John Duffy, Chief
Boiler Inspector

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

November 10, 1980

IE Information Notice No. 80-41: FAILURE OF SWING CHECK VALVE IN THE DECAY
HEAT REMOVAL SYSTEM AT DAVIS-BESSE UNIT
NO. 1

Description of Circumstances:

On October 9, 1980 the resident inspector at the Davis-Besse facility was informed that the licensee had performed leak rate tests and identified excessive leakage through Decay Heat Removal System check valve CF-30. Valve CF-30 is the inboard one of two in-series check valves that is used to isolate the reactor coolant system from the low pressure decay heat removal system. On further investigation the licensee found that the valve disc and arm had separated from the valve body and was lodged just under the valve cover plate. The two 2 5/8 x 5/8 inch bolts and locking mechanism for the bolts that holds the arm to the valve body were missing and have not been located. The CF-30 valve is a 14-inch swing check valve manufactured by Velan Valve Corporation. The cause of the failure has not been identified.

The test that was in progress when the failure was identified was being conducted in response to a letter from Darrell G. Eisenhut, Office of Nuclear Reactor Regulations to all Light Water Reactor Licensees, LWR Primary Coolant System Pressure Isolation Valves, February 23, 1980.

This IE Information Notice is provided as an early notification of a possibly significant matter that is still under review by the NRC staff. It is expected that recipients will review the information for possible applicability to their facilities. No specific action or response is requested at this time. If NRC evaluations so indicate, further licensee actions may be requested or required.

No written response to this IE Information Notice is required. If you have any questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

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