

CYCLE 4 STARTUP TEST REPORT

Cycle 4 operations commenced August 8, 1980 with the withdrawal of the first control rod. The startup test program was conducted from July 18, 1980 through October 25, 1980, in accordance with Reactor Analyst Procedure (RAP) 7.1.17, titled Refuel Startup Program, Revision 3. When reference is made to values of core thermal power and core flow, these are nominal values rather than exact percentages.

CONTROL ROD DRIVE TESTS

Control rod drive coupling checks were satisfactorily completed on August 17, 1980. In addition, the insert and withdrawal times for rods were checked and adjusted as required.

Prior to reaching 40% rated core thermal power, control rod scram time testing was conducted in accordance with RAP 7.3.10 titled Control Rod Scram Time Evaluation, Revision 3. This test requires that each control rod be scrambled from position 48 (full out) with reactor pressure > 950 psig. The results of these tests are tabulated below.

Results:

<u>Control Rod Notch Position Observed</u>	<u>Technical Specification (Seconds)</u>	<u>Average of 137 Rods (Seconds)</u>
46	0.338	0.312
38	0.923	0.719
24	1.992	1.452
04	3.554	2.569

The average of the scram insertion times for the three fastest operable control rods for all groups of four control rods a in two-1 -two array were less than the maximum allowed by the technical specifications.

SHUTDOWN MARGIN DEMONSTRATION

A shutdown margin (SDM) demonstration was performed August 8, 1980 in accordance with Reactor Analyst Procedure 7.3.9 Revision 2. The required SDM was $0.38\% \Delta k + R + \text{temperature defect}$. The fuel vendor has calculated that the value of R is 0.15%. After calculating the temperature defect and uncertainty, the required SDM was determined to be $0.78\% \Delta k$. Sufficient control rods were withdrawn to demonstrate a SDM of $0.80\% \Delta k$. The calculated SDM based on the in-sequence critical data was 1.69%.

INSEQUENCE CRITICAL

Based on data supplied by the fuel vendor, the first critical was estimated to be notch 10 of the eleventh rod of group 2 with a reactor water temperature of 145°F . The actual data was group 2, rod 14, notch 08, reactor water temperature 155°F , period 89 seconds, indicating good agreement between predicted and actual data.

REACTIVITY ANOMALY CHECK

A comparison of the expected and actual control rod density was performed at 100% core thermal power (CTP) and 100% rated core flow. The control rod inventory was 418 notches which was in close agreement with the predicted value of 421 notches. The $\pm 1\%$ reactivity boundaries were 80 to 720 notches.

POWER DISTRIBUTION MEASUREMENTS

Core power distribution was monitored throughout the startup using the process computer. Following significant changes in control rod pattern and power level, a complete power distribution measurement was performed using the Traversing In-core Probe (TIP) system. Core parameters were maintained within technical specification limits.

TIP REPRODUCIBILITY

Four successive traces were run in the common channel for each TIP machine. Computer reduction of the data calculated a random noise of 1.46% and a total uncertainty of 3.14%, well below the 8.7% assumed by the vendor in the statistical analysis performed for the licensing topical report (NEDG-24011-P-A) for the reload fuel application.

CORE POWER SYMMETRY

Core power symmetry was checked at 25%, 50%, 75% and 100% CTP. Mirror symmetric fuel assemblies checked at 100% CTP and 100% core flow using the process computer indicate a maximum difference of less than 10%.

CORE LOADING

A copy of the final core loading is attached as Figure 1. Irradiated fuel returned to the core is designated EA or LJ5 or LJ6 or LJB. There were 160 new fuel assemblies, designated LJM, (136 bundles with 2.82 w/o U-235, 24 bundles with 2.65 w/o U-235) loaded during the refueling outage. The new fuel was of the P8 x 8 R design with an active fuel length of 150 inches and received 100 mil channels.

Figures 2a - 2d show the approximate irradiated bundle average exposure following refueling. A zero indicates a new fuel assembly.

The new fuel assemblies contain burnable poison in the form of GdO_3 . The concentration and location is proprietary to the fuel vendor.

Figure 3 shows the rod sequence control system (RSCS) designations for the A and B rod withdrawal sequences.

During the refueling operation, each fuel move was checked by an individual, other than the operator performing the move, and verified independently by a third individual. Two lines of communication were established between the refuel bridge and the control room. Following loading, a core verification was conducted (and video-taped and examined later by quality assurance personnel) to verify the correct placement and orientation of each assembly.

ADDITIONAL TESTS

1. Tests were performed in accordance with RAP 7.1.17 and F-ST-5C to verify that there is approximately one decade overlap between the source range monitor and intermediate range monitor (IRM) systems, and between the IRM and average power range monitor (APRM) systems.
2. Reactor core isolation cooling and high pressure coolant injection flow rate tests were performed in accordance with F-ST-24C and F-ST-4B and demonstrated compliance with the technical specifications.
3. Both the rod worth minimizer and rod sequence control system functioned properly during the startup test program.
4. Since some TIP tubing were replaced, the TIP alignment and logic limits were checked and adjusted as required.
5. New computer software was installed during the outage. Extensive testing was performed prior to and during the startup in accordance with the vendor's recommendations.

6. The APRM system was calibrated to core thermal power and satisfactorily tracked power changes.
7. Heat balances were calculated manually and used to verify the process computer calculations.
8. Process computer calculations of fuel assembly parameters, maximum average planar linear heat generation rate, minimum critical power ratio, and maximum fraction limiting power density compared satisfactorily with results obtained using off-line computer calculations.
9. RAP 7.3.18, "Pressure Regulator Tests," was performed satisfactorily when it was verified that an induced pressure transient of 10 psi was controlled by the electro-hydraulic control system pressure regulator. In addition, transfer from the primary to back-up pressure regulator was demonstrated following a simulated failure of the primary regulator.
10. RAP 7.3.7, "Core Flow Evaluation and Indication Calibration," was performed at 75%, 89% and 96% CTP. The results of the September 12, 1980 tests at 96% CTP showed a calculated core flow of 72.2 Mlb/hr while the indicated core flow was 77.1 Mlb/hr.

After adjustment of the amplifier gains, the calibration was repeated September 16, 1980. Calculated core flow was 75.3 Mlb/hr with an indicated core flow of 76.4 Mlb/hr.

FIGURE 1

ISER/ISRBB FUEL SERIAL NUMBERS

[illegible]

1 3 5 7 9 11 13 15 17 19 21 23 25 27 29 31 33 35 37 39 41 43 45 47 49 51

10/16/80

EBUN	BUNDLE EXPOSURES				FITZPATRICK	1	QUADRANT 1				1356	10/16/80
1	3	5	7	9	11	13	15	17	19	21	23	25
52									19474.	19425.	18769.	18504. 18291. 52
50								19611.	6876.	0. 7101.	0. 7641.	50
48						19959.	19632.	18985.	8313.	19321.	7948.	17952. 6878. 0. 48
46				19860.	7878.	0. 7471.	0. 7484.	0. 7615.	0. 12854.	46		
44				19330.	7965.	7137.	13953.	7786.	13814.	0. 13890.	6863.	13680. 0. 44
42				18875.	0. 14168.	0. 12833.	0. 12551.	0. 12501.	0. 13795.	42		
40				18408.	7225.	7179.	12021.	0. 13222.	6421.	14031.	6496.	18490. 0. 40
38				19319.	7716.	0. 13043.	0. 14053.	0. 11914.	0. 11083.	0. 17541.	38	
36				19010.	7466.	18410.	7715.	0. 13011.	6078.	12100.	0. 18432.	6043. 18215. 0. 36
34				19368.	0. 7443.	0. 13835.	0. 14055.	0. 17474.	0. 11450.	0. 12012.	34	
32				18634.	6861.	19122.	5761.	6996.	11894.	6467.	11531.	5770. 11062. 6562. 14561. 4843. 32
30				18414.	0. 6476.	0. 13427.	0. 18428.	0. 19333.	0. 14305.	0. 16133.	30	
28				19297.	7128.	0. 13026.	0. 15182.	0. 18500.	0. 12401.	4741.	16351.	14097. 28
	1	3	5	7	9	11	13	15	17	19	21	23 25

Figure 2a

10/16/80

EBUN		BUNDLE EXPOSURES			FITZPATRICK		1	QUADRANT 2		1356	10/16/80			
27	29	31	33	35	37	39	41	43	45	47	49	51		
52	17904.	18075.	18675.	19725.	19968.							52		
50	7649.	0.	7093.	0.	6876.	19911.						50		
48	0.	6881.	17941.	7951.	19418.	8326.	19481.	19449.	19670.			48		
46	12858.	0.	7621.	0.	7483.	0.	7472.	0.	7878.	19940.		46		
44	0.	13662.	6865.	13887.	0.	13795.	7798.	13932.	7142.	7963.	18946.	44		
42	1775.	0.	12498.	0.	12541.	0.	12831.	0.	14152.	0.	18847.	42		
40	0.	18323.	6493.	14019.	6416.	13224.	0.	12001.	7181.	7223.	18434.	40		
38	17529.	0.	11065.	0.	11914.	0.	14048.	0.	13046.	0.	7717.	19251.	38	
36	0.	18416.	6051.	18488.	0.	12095.	6083.	13013.	0.	7713.	18441.	7456.	19598.	36
34	12003.	0.	11459.	0.	18402.	0.	14055.	0.	13830.	0.	7442.	0.	19635.	34
32	4841.	14564.	6559.	11057.	5765.	11525.	6465.	11899.	7001.	5767.	18995.	6863.	18003.	32
30	15725.	0.	14298.	0.	18867.	0.	18418.	0.	13422.	0.	6475.	0.	18315.	30
28	14108.	16694.	4739.	12396.	0.	18668.	0.	15172.	0.	13041.	0.	7127.	19536.	28
27	29	31	33	35	37	39	41	43	45	47	49	51		

Figure 2b

10/16/80

EBUN		BUNDLE EXPOSURES				FITZPATRICK				1	QUADRANT 3 1356 10/16/80			
27	29	31	33	35	37	39	41	43	45	47	49	51		
26	14103.	16568.	4742.	12403.	0. 18977.	0. 15205.	0. 13031.	0. 7121.	19285.				26	
24	16180.	0. 14314.	0. 18977.	0. 17089.	0. 13440.	0. 6468.	0. 18142.						24	
22	4851.	14581.	6555.	11046.	5768.	11545.	6468.	11890.	7000.	5770.	18842.	6867.	18393.	22
20	12010.	0. 11474.	0. 18454.	0. 14069.	0. 13826.	0. 7435.	0. 19663.						20	
18	0. 18421.	6055.	18436.	0. 12098.	6078.	12986.	0. 7717.	18530.	7475.	19013.			18	
16	17768.	0. 11083.	0. 11903.	0. 14065.	0. 13046.	0. 7713.	19028.						16	
14	0. 17958.	6496.	14018.	6416.	13212.	0. 12015.	7177.	7218.	18476.				14	
12	13781.	0. 12489.	0. 12552.	0. 12842.	0. 14172.	0. 18794.							12	
10	0. 13666.	6867.	13904.	0. 13801.	7787.	13922.	7142.	7966.	18738.				10	
8	12854.	0. 7621.	0. 7478.	0. 7474.	0. 7872.	19947.							8	
6	0. 6880.	17911.	7950.	19359.	8314.	18897.	19218.	19779.					6	
4	7642.	0. 7077.	0. 6875.	19604.									4	
2	18146.	18543.	18761.	19695.	19945.								2	
27	29	31	33	35	37	39	41	43	45	47	49	51		

Figure 2c

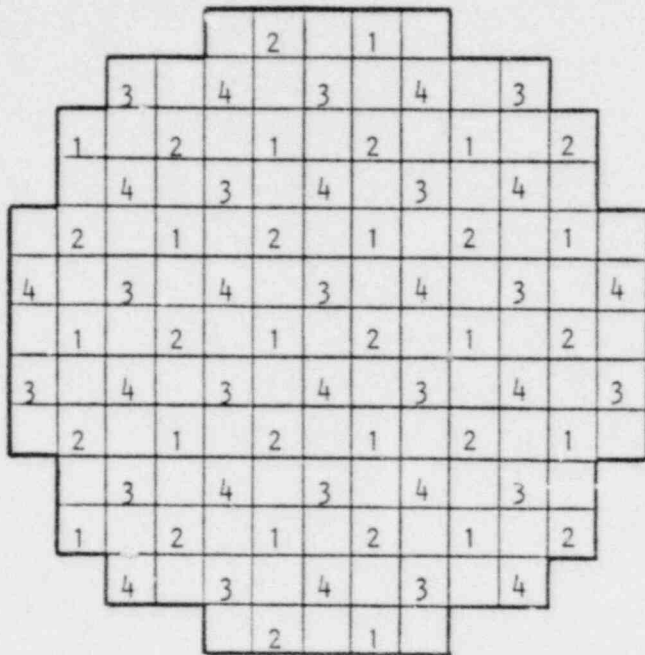
10/16/80

EBUN	BUNDLE EXPOSURES				FITZPATRICK	1	QUADRANT 4				1356	10/16/80	
	1	3	5	7	9	11	13	15	17	19	21	23	25
26	19547.	7136.	0. 13034.	0. 15185.	0. 19132.	0. 12398.	4742.	16944.	14109.	26			
24	18419.	0. 6473.	0. 13433.	0. 17095.	0. 18962.	0. 14300.	0. 16378.	24					
22	18730.	6859.	18893.	5773.	6994.	11887.	6467.	11541.	5767.	11068.	6564.	14566.	4848. 22
20	19657.	0. 7441.	0. 13826.	0. 14071.	0. 17477.	0. 11461.	0. 11996.	20					
18	19086.	7461.	18498.	7711.	0. 12998.	6076.	12093.	0. 18552.	6049.	18184.	0. 18		
16		18771.	7719.	0. 13041.	0. 14058.	0. 11921.	0. 11076.	0. 17804.	16				
14			18781.	7228.	7182.	12022.	0. 13210.	6414.	14005.	6495.	18008.	0. 14	
12			18461.	0. 14160.	0. 12639.	0. 12548.	0. 12498.	0. 13786.	12				
10			18862.	7961.	7141.	13926.	7789.	13808.	0. 13900.	6861.	13673.	0. 10	
8				19172.	7875.	0. 7468.	0. 7482.	0. 7613.	0. 12851.	8			
6					19803.	19216.	19504.	8317.	19394.	7950.	17945.	6879.	0. 6
4								19399.	6876.	0. 7081.	0. 7627.	4	
2										20193.	19557.	18766.	18645. 18479. 2
	1	3	5	7	9	11	13	15	17	19	21	23	25

Figure 2d

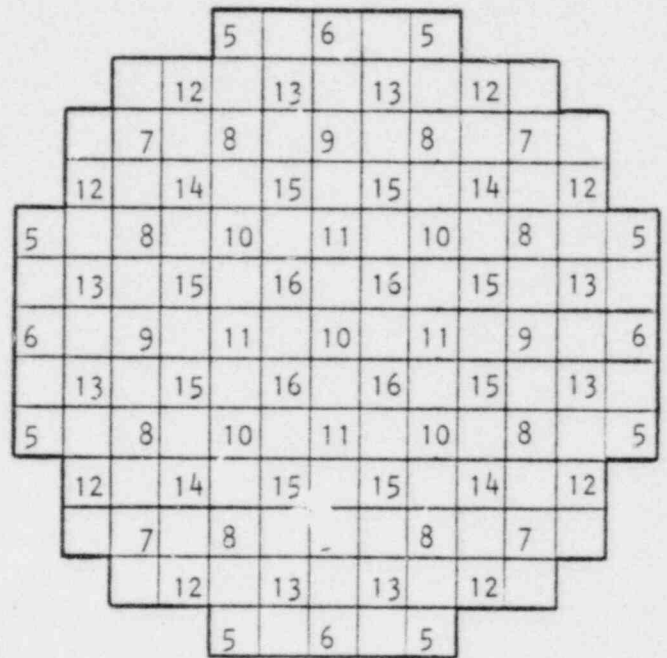
ROD SEQUENCE CONTROL SYSTEM GROUP DESIGNATIONS

A SEQUENCE



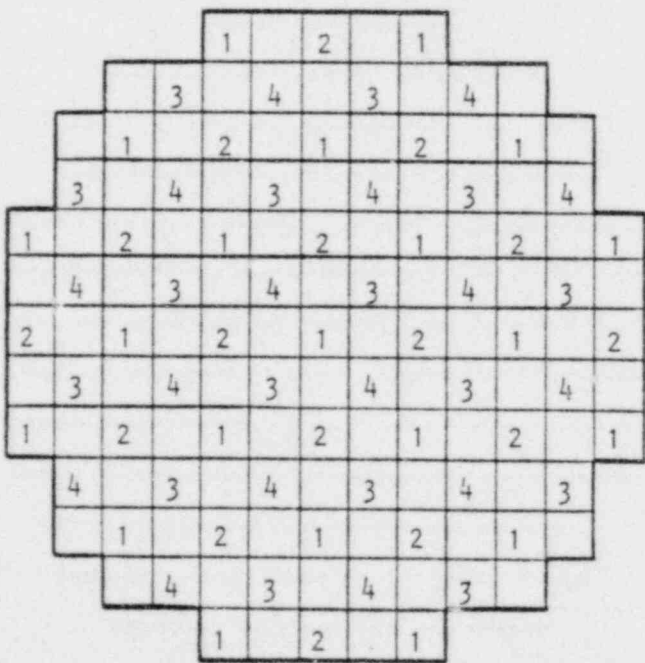
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51
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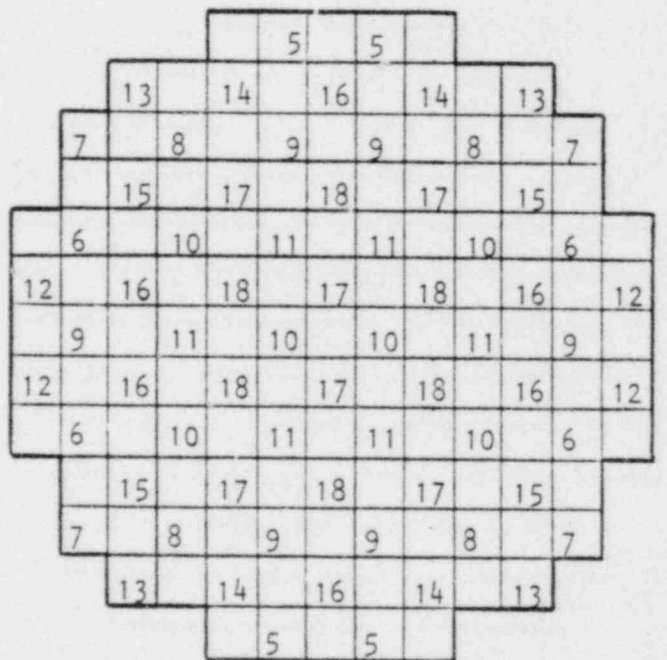
02 06 10 14 18 22 26 30 34 38 42 46 50

B SEQUENCE



02 06 10 14 18 22 26 30 34 38 42 46 50

51
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02 06 10 14 18 22 26 30 34 38 42 46 50

Fig. 3