NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

EXHIBIT A

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | F L C R P 3 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | | (5)
LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T
0 1 | REPORT SOURCE L (6) | DOCKET NUMBER 0 5 0 - 0 3 0 2 (7) | EVENT DATE 1 0 0 8 8 0 (8) | REPORT DATE 1 1 0 5 8 0 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | At 1126 during Reactor startup after a Reactor trip it was discovered by
0 3 | Reactor Coolant System sampling that the Dose Equivalent I-131 exceeded 1.0
0 4 | microcuries per gram. This created an event contrary to T.S. 3.4.8. Redundancy
0 5 | N/A. There was no effect upon the general public health or safety. This is
0 6 | the eighteenth event reported under this specification.
0 7 |
0 8 |

0 9 | SYSTEM CODE Z Z (11) | CAUSE CODE X (12) | CAUSE SUBCODE Z (13) | COMPONENT CODE Z Z Z Z Z Z (14) | COMP. SUBCODE Z (15) | VALVE SUBCODE Z (16)

(17) LER/RO REPORT NUMBER | EVENT YEAR 8 0 | — | SEQUENTIAL REPORT NO. 0 4 2 | / | OCCURRENCE CODE 0 3 | REPORT TYPE L | — | REVISION NO. 0

ACTION TAKEN X (18) | FUTURE ACTION Z (19) | EFFECT ON PLANT Z (20) | SHUTDOWN METHOD Z (21) | HOURS 0 0 0 0 (22) | ATTACHMENT SUBMITTED Y (23) | NPRD-4 FORM SUB. N (24) | PRIME COMP. SUPPLIER Z (25) | COMPONENT MANUFACTURER Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The cause of this event is attributed to leaking fuel pins and an anticipated
1 1 | Iodine transient following a Reactor Coolant System transient. Reactor Coolant
1 2 | purification flow was increased; Dose Equivalent I-131 was less than 1.0 microcuries
1 3 | per gram at 1250.
1 4 |

1 5 | FACILITY STATUS C (28) | % POWER 0 9 8 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY B (31) | DISCOVERY DESCRIPTION (32) Technician Analysis

1 6 | ACTIVITY RELEASED Z (33) | CONTENT OF RELEASE Z (34) | AMOUNT OF ACTIVITY (35) NA | LOCATION OF RELEASE (36) NA

PERSONNEL EXPOSURES
1 7 | NUMBER 0 0 0 (37) | TYPE Z (38) | DESCRIPTION (39) NA

PERSONNEL INJURIES
1 8 | NUMBER 0 0 0 (40) | DESCRIPTION (41) NA

LOSS OF OR DAMAGE TO FACILITY (43)
1 9 | TYPE Z (42) | DESCRIPTION NA

PUBLICITY
2 0 | ISSUED N (44) | DESCRIPTION (45) NA | NRC USE ONLY

Name of Preparer: Victor A. Hernandez PHONE (904) 795-6486

(SEE ATTACHED SUPPLEMENTARY INFORMATION SHEET)

8011110 328

SUPPLEMENTARY INFORMATION

Report No.: 50-302/80-042/03L-0

Facility: Crystal River Unit #3

Report Date: 5 November 1980

Occurrence Date: 8 October 1980

Identification of Occurrence:

Primary Coolant Dose Equivalent I-131 was greater than 1.0 microcuries per gram contrary to Technical Specification 3.4.8.a.

Conditions Prior to Occurrence:

Mode 1 power operation (98%)

Description of Occurrence:

At 1126 during Reactor startup after a Reactor trip it was discovered that Dose Equivalent I-131 exceeded 1.0 microcuries per gram. This determination was made by analysis of a Reactor Coolant sample drawn at 0850. Redundancy was not applicable.

Designation of Apparent Cause:

The cause of this event is attributed to leaking fuel pins and an anticipated Iodine transient following a Reactor Coolant System transient.

Analysis of Occurrence:

There was no effect upon the general public health or safety.

Corrective Action:

Reactor Coolant purification flow was increased to aid cleanup. Dose Equivalent I-131 was less than 1.0 microcuries per gram at 1250.

Failure Data:

This is the eighteenth event reported under this specification.

/rc

Reactor Power History of Prior

Forty-eight Hours

Item I

Event Date: 8 October 1980

10, 6,80

HOUR	MWTH	
1	97.23	98.60
2	97.19	98.40
3	97.23	98.40
4	97.19	98.40
5	95.47	96.60
6	96.86	98.00
7	99.35	100.10
8	99.35	100.10
9	99.39	100.10
10	99.35	100.20
11	99.31	100.10
12	99.31	100.10
13	99.43	100.20
14	99.43	100.20
15	99.39	100.20
16	99.31	100.20
17	99.23	100.50
18	99.14	100.70
19	98.78	100.50
20	98.74	100.40
21	98.41	100.10
22	97.92	99.60
23	97.89	99.40
24	97.96	99.40

98.45%

DATE 10, 7,80

HOUR	GMWE (E710) °/°FP	TURB G (T856) BTU/KWH	MWTH (P753) °/°FP	NI (P723) °/°FP	RATIO NI/MT °/°FP	RATIO ME/MT °/°FP
1	95.81	10007	98.04	99.30	1.013	.977
2	95.80	10543	98.04	99.10	1.011	.977
3	95.79	10005	98.00	99.00	1.010	.977
4	95.88	9995	98.00	98.90	1.009	.978
5	91.23	9861	91.72	93.80	1.023	.995
6	95.18	10002	97.31	98.20	1.009	.978
7	97.45	9980	99.51	99.90	1.004	.979
8	97.38	9991	99.55	99.90	1.004	.978
9	97.37	9992	99.55	100.10	1.006	.978
10	97.44	9985	99.55	100.20	1.007	.979
11	97.36	10000	99.63	100.40	1.008	.977
12	97.32	10027	99.88	100.70	1.008	.974
13	95.98	10168	99.88	100.80	1.009	.961
14	97.36	10018	99.80	100.80	1.010	.976
15	97.36	9995	99.55	100.50	1.010	.978
16	97.39	9990	99.55	100.30	1.008	.978
17	97.36	9996	99.55	100.30	1.008	.978
18	97.36	9993	99.55	100.20	1.007	.978
19	97.32	9991	99.47	100.00	1.005	.978
20	96.43	9992	98.53	99.10	1.006	.979
21	96.25	9987	98.29	98.90	1.006	.979
22	96.25	9978	98.21	98.70	1.005	.980
23	96.21	9977	98.16	98.90	1.007	.980
24	96.46	9978	98.41	99.00	1.006	.980

AVERAGE DAILY GENERATOR GROSS 824.97 MWH(E)
AVERAGE DAILY THERMAL POWER 2419.04 MWH(T)
AVERAGE DAILY TURBINE GROSS HEAT RATE 10019 BTU/KWH
AVERAGE DAILY MWTH POWER 98.656 °/°FP
AVERAGE DAILY NUCLEAR INST. POWER 99.458 °/°FP

RATIO OF NI TO MWTH = 1.008

DATE 10, 8,80

HOUR	GMWE (E710) °/°FP	TURB G (T856) BTU/KWH	MWTH (P753) °/°FP	NI (P723) °/°FP	RATIO NI/MT °/°FP	RATIO ME/MT °/°FP
1	13.08	0	14.93	13.70	.918	.876
2	0.00	0	.00	0.00	0.000	0.000
3	0.00	0	.00	0.00	0.000	0.000
4	0.00	0	.82	0.00	0.000	0.000
5	0.00	0	.41	0.00	0.000	0.000
6	.16	196	7.59	10.00	1.318	.022
7	18.56	15182	25.98	31.00	1.193	.714
8	51.26	11068	56.36	60.60	1.075	.910
9	84.34	9908	84.99	97.50	1.147	.992
10	93.64	9945	95.19	98.00	1.030	.984
11	93.13	9939	94.62	98.40	1.040	.984
12	93.37	9938	94.82	98.40	1.038	.985
13	93.47	9953	95.02	98.60	1.038	.984
14	93.45	9959	95.07	98.20	1.033	.983
15	93.57	10526	95.11	98.30	1.034	.984
16	93.58	9958	95.19	98.30	1.033	.983
17	93.95	9959	95.19	98.50	1.035	.987
18	94.02	9965	95.72	98.50	1.029	.982
19	94.63	9968	96.41	98.80	1.025	.982
20	95.27	9976	97.15	99.00	1.019	.981
21	95.78	9978	97.72	98.90	1.012	.980
22	96.02	9983	98.04	99.00	1.010	.979
23	96.23	9985	98.29	99.00	1.007	.979
24	96.42	9973	98.37	99.10	1.007	.980

AVERAGE DAILY GENERATOR GROSS 564.28 MWH(E)
AVERAGE DAILY THERMAL POWER 1668.33 MWH(T)
AVERAGE DAILY TURBINE GROSS HEAT RATE 7765 BTU/KWH
AVERAGE DAILY MWTH POWER 68.040 °/°FP
AVERAGE DAILY NUCLEAR INST. POWER 70.492 °/°FP

RATIO OF NI TO MWTH = 1.036

Fuel Burnup by Core Region

Item 2

Event Date: 8 October 1980

ITEM 2

The burnup was claculated at 45.2 EFPD for the three (3) enrichment regions.

BATCH	NUMBER of FA	BURNUP	
B	9	MWD/MTU	19272
C	60	MWD/MTU	17096
D	52	MWD/MTU	6105
E	56	MWD/MTU	1110
AVE.		MWD/MTU	8920

CLEANUP FLOW HISTORY

ITEM 3

EVENT DATE: 8 October 1980

ITEM 3

Cleanup flow history forty-eight (48) hours prior to the first sample in which the limit was exceeded indicated a letdown flowrate of fifty (50) gpm.

DATE	TIME	FLOWRATE
10/6/80	0050	50 gpm
10/7/80	0030	50 gpm

Degas Operations

Item 4

Event Date: 10/8/80

DATE	TIME	Location
10/6/80	2142	WGDT-C

Time Duration When DEI-131 Exceeded 1.0

μ Ci/gram and I-131 Analysis Results

Item 5

Event Date: 8 October 1980

ITEM 5

As per Technical Specification 3.4.8, the four (4) hour sampling frequency as depicted on the table below was initiated at 0850 on 8 October 1980 and the Dose Equivalent I-131 was 1.05 microcuries per gram. The four hour sampling frequency was terminated at 1650 on 8 October 1980 when the DEI-131 was determined to be .485 microcuries per gram. The time duration when the specific activity of primary coolant exceeded 1.0 microcuries per gram Dose Equivalent is less than four (4) hours.

DATE	TIME	DEI-131 (μCi/gram)
10/8/80	0850	1.05
10/8/80	1250	.665
10/8/80	1650	.485