



# Public Service Company <sup>of</sup> Colorado

16805 ROAD 19½  
PLATTEVILLE, COLORADO 80651

October 14, 1980  
Fort St. Vrain  
Unit No. 1  
P-80366

Mr. Karl V. Seyfrit, Director  
Nuclear Regulatory Commission  
Region IV  
Office of Inspection and Enforcement  
611 Ryan Plaza Drive  
Suite 1000  
Arlington, Texas 76012

Reference: Facility Operating License  
No. DPR-34

Docket No. 50-267

Dear Mr. Seyfrit:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/80-50, Final, submitted per the requirements of Technical Specification AC 7.5.2(b)2 and AC 7.5.2(b)3.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/80-50.

Very truly yours,

*Don Warembourg*  
Don Warembourg  
Manager, Nuclear Production

DW/cls

Enclosure

cc: Director, MIPC

*A002  
5/11*

8010210 652

*5*

REPORT DATE: October 14, 1980  
Determined  
OCCURRENCE DATE: September 15, 1980

REPORTABLE OCCURRENCE 80-50  
ISSUE 0  
Page 1 of 3

FORT ST. VRAIN NUCLEAR GENERATING STATION  
PUBLIC SERVICE COMPANY OF COLORADO  
16805 WELD COUNTY ROAD 19 1/2  
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/80-50/03-L-0

Final

IDENTIFICATION OF  
OCCURRENCE:

Two hydraulic snubbers installed on Class I piping had not been identified as Class I, nor included in Table 4.3.10-1 of LCO 4.3.10 and were therefore not routinely tested to verify operability per SR 5.3.8.

Because compliance with LCO 4.3.10 cannot be verified for previous reactor operation, this is reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)2 and AC 7.5.2(b)3. Reportability determined September 15, 1980.

EVENT  
DESCRIPTION:

Engineering personnel performing an audit of 2 1/2" and larger piping identified hydraulic snubbers BFS-154E and BFS-166 which, although installed on Class I piping, were not included in the list of Class I snubbers contained in Fort St. Vrain Technical Specification LCO 4.3.10, Table 4.3.10-1. Because of the omission of BFS-154E and BFS-166 from Table 4.3.10-1, the operability criteria and testing requirements of the Technical Specifications had not previously been applied to these snubbers. These two snubbers are the only omissions identified to date.

Although the plant was in a shutdown condition for scheduled maintenance and surveillance testing at the time of this discovery, numerous periods of reactor operation have occurred since these snubbers were initially installed.

CAUSE  
DESCRIPTION:

BFS-154E and BFS-166 were apparently omitted from the list of Class I snubbers contained in Table 4.3.10-1 due to an oversight on the part of personnel who performed the initial analysis of piping supports.

CORRECTIVE  
ACTION:

The following corrective actions have been taken with regard to this occurrence:

- 1) The snubbers were inspected per the criteria of SR 5.3.8a-X and found to be acceptable. In addition, the snubbers were functionally tested in accordance with approved maintenance procedures. This meets the intent of Surveillance Requirement 5.3.8b-R.
- 2) A revision has been made to include BFS-154E and BFS-166 in Surveillance Test SR 5.3.8a-X.
- 3) A revision to LCO 4.3.10, Table 4.3.10-1 to add these two new snubbers will be submitted per LCO 4.3.10(e).

No further corrective action is anticipated or required.

Prepared By:

Cathy C. Hirsch  
Cathy C. Hirsch  
Technical Services Technician

Reviewed By:

J. W. Gahm  
J. W. Gahm  
Technical Services Supervisor

Reviewed By:

Frank M. Mathie  
Frank M. Mathie  
Operations Manager

Approved By:

Don Warembourg  
Don Warembourg  
Manager, Nuclear Production