

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-10

DRESDEN NUCLEAR POWER STATION UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11
License No. DPR-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated December 3, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility License No. DPR-2 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 30.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by:
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Reactor Licensing

Attachment:
Change No. 30 to the
Technical Specifications

Date of Issuance: JUL 12 1975

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Y. S. S. S. S.						
DATE OF						

ATTACHMENT TO LICENSE AMENDMENT NO. 11

CHANGE NO. 30 TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-2

Delete pages 63 and 64 from the Technical Specifications and insert the attached replacement pages numbered 63, 63a, and 64. The changes on the revised pages are shown by marginal lines.

9. From and after the date that one of a pair of redundant motor operated valves is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding 48 hours unless the valve is sooner made operable provided that the remaining valve is operable.

10. From and after the date that one Unit 1 screen wash pump is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding 48 hours unless the pump is sooner made operable.

11. Until the HPCI system which is discussed in Dresden Station Special Report No. 27 is operational, the following additional conditions shall be met.

a. During reactor power operation with reactor pressure greater than 140 psig, at least one primary reactor feedwater pump will be operable with the source of power from transformer TR-12.

b. During reactor power operation with reactor pressure greater than 140 psig, the emergency reactor feedwater pump will be operable.

c. From and after the date that condition a or b is not met, continued reactor operation is permissible only during the succeeding 7 days unless the requirements are sooner met.

10. Unit 1 Screen Wash Pumps

a. Pump operability once/month

b. Motor valve operated once/month

11. Primary Reactor Feedwater System

a. The operability and source of power of the primary reactor feedwater pumps will be logged at least daily.

b. The emergency feedwater pump will be started at least every 30 days to verify operability.

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12. If the requirements of 3.5.A cannot be met, an orderly shutdown of the reactor shall be initiated and the reactor shall be in the cold shutdown condition within 24 hours.

Bases:

- 3.5 Dresden Unit 1 is provided with a spray type, emergency core cooling system for use in the event of a serious loss-of-coolant accident. Supplement E to Change No. 17 provides an analysis which shows that the core spray system in conjunction with the isolation condenser and feedwater system limits peak clad temperatures to less than 2300°F. Metal water reactions are limited to less than 1% by the above systems.

The core spray system piping enters through the reactor vessel head and therefore the system is only operable when the reactor head is in place. The core spray system gets its water from the station fire main system. This system obtains its water from the Unit 1 screen wash pumps, Unit 2/3 service water pumps or the Unit 1 or Unit 2 diesel driven fire pumps. The latter pumps are for emergency use if off-site power is lost.

The allowable repair times were arrived at by a consideration of the redundancy of components and reliability of performing the intended function if required.

The core spray system has 3 pumps, any 2 of which are capable of supplying the required flow. Thus, if one pump is out of service, no additional failure could be tolerated. Thus, an out of service time of 48 hours is specified. This is true for other components

such as redundant valves, post incident pumps, etc.

The periodic testing intervals are specified to provide a monthly test of all pumps and valves in the post incident (core and containment) cooling system. These tests are to verify operability only and not to determine capability of the component to perform its intended function. Capability tests, such as flow tests of the core spray pumps, are performed each refueling outage. This interval is adequate since no changes to the system are foreseen following installation.

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The operability of the primary reactor feedwater system provides added assurance that feedwater will be available to supplement the core spray system for small to intermediate size loss-of-coolant accidents.