

# NSP

NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

February 9, 1981

Director of Nuclear Reactor Regulation  
U S Nuclear Regulatory Commission  
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT  
Docket No. 50-282 License No. DPR-42  
50-306 DPR-60

Request for Extension of Completion Dates for Approved  
Fire Protection Modifications

By letter dated November 24, 1980 the Staff provided guidance on implementation of the requirements of Appendix R to 10 CFR Part 50. The purpose of this letter is to request extension of the completion dates specified in 10 CFR Part 50, Section 50.48(d) for several approved fire protection modifications.

Section 50.48(d) requires a completion date of February 17, 1981 for the following approved fire protection modifications which will not be completed by that date:

<u>Fire Protection SER Item Number</u>	<u>Description</u>
3.1.1(2) & 3.1.1(4)	Upgrade Fire Wall Seals
3.1.5	Turbine Building Structural Steel Protection
3.2.6	Fire Pump Enclosure

The best estimated completion date, the current status, the reason for extension, and an evaluation of the impact on safe plant operation if the extension is granted follows for each of these modifications:

Upgrade Fire Wall Seals, Items 3.1.1(2) and 3.1.1(4)

An extension is requested to November 17, 1981 for this modification. Approximately 2000 penetrations were identified which required retrofitting with qualified seals. Of the areas identified in the Fire Protection SER for seal upgrading, 95 penetrations have yet to be completed; 15 in Unit No. 1 and 80 in Unit No. 2. Fifty-one other penetrations for which good fire protection practice indicates upgrading is needed also remain to be completed.

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Seals remaining to be completed are in the vicinity of steam or feed-water piping or below critical switchgear cabinets. These will be completed during the Unit No. 1 refueling outage scheduled to begin in September, 1981 or the Unit No. 2 outage scheduled to begin later this month.

Existing unqualified seals, fire detection instruments, and automatic and manual fire suppression equipment are adequate to limit the spread of fire in safety related areas of the plant while seal upgrading is being completed.

### Turbine Building Structural Steel Protection, Item 3.1.5

An extension is requested to the end of the Unit No. 2 1981 refueling outage (now scheduled to begin later this month).

Structural steel coating on the Unit No. 1 side of the turbine building is completed. Coating of steel above the 727-foot level on the Unit No. 2 side cannot be completed until the refueling outage. Personnel safety considerations and the adverse impact on curing of the material prevent application of the coating with Unit No. 2 in operation.

The lack of safe shutdown equipment in this area, combined with the structural protection already in place, justify continue operation of Unit No. 2 until this task is completed.

### Fire Pump Enclosure, Item 3.2.6

An extension is requested for this item until June 1, 1981.

A fire rated door and frame for the enclosure is still on order. The door and frame originally ordered did not meet specifications and these components have been reordered from a local supplier. All necessary cable rerouting has been completed.

Following receipt of the door and frame (expected next month), completion of this enclosure and the related day tank curb (item 3.1.4(11)) should be completed within an additional eight weeks (allowing sufficient time for concrete to cure).

Continued safe operation of the plant is assured by the redundancy in supplies to the plant fire suppression water system. In addition to the diesel-driven fire pump and the motor-driven pump, fire suppression water can be supplied by the screen wash pump or the safeguards grade cooling water system.

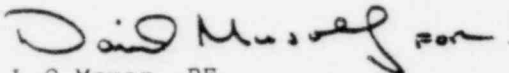
In addition to the items discussed above, Appendix R requirements contained in Sections III.G, III.J, and III.O have been applied to all plants by the Commission regardless of the results of previous NRC Staff evaluations and the conclusions of the Fire Protection Safety Evaluation Report. Section III.J, "Emergency Lighting," is believed to be satisfied by the existing plant design. Section III.O, "Oil Collection System for Reactor Coolant Pump," is under

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review by our architect-engineer who will analyze the existing oil collection system. The requirements of Section III.G, "Fire Protection of Safe Shutdown Capability," supersede the requirements of the Fire Protection Safety Evaluation Report contained in items 3.1.12 and 3.2.1. Modifications required by the Fire Protection SER for these items (identified in our letter of October 22, 1979) will be reevaluated in accordance with Section III.G of Appendix R. They will be completed, if found to still be appropriate, in accordance with the schedule presented in 10 CFR 50 Section 50.48(c). A schedule for completing these modifications and a description of planned alternate shutdown modifications will be submitted for NRC review by March 19, 1981 as required by Appendix R.

Please contact us if you have any questions concerning this information.



L O Mayer, PE  
Manager of Nuclear Support Services

LOM/DMM/jh

cc J G Keppler  
NRC Resident Inspector  
G Charnoff