



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 46
License No. DPR-35

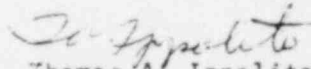
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Boston Edison Company (the licensee) dated September 29, 1980 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 46, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license Amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical Specifications

Date of Issuance: February 2, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 46

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Revise Appendix A as follows:

Remove the following pages and insert identically numbered pages:

208
208a
209
210
210a
212
213
216
217

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

The Nuclear Operations Manager shall be accountable for overall facility operation. In his absence the Deputy Nuclear Operations Manager or an alternate designated in writing by the Nuclear Operations Manager shall assume this responsibility.

6.2 ORGANIZATION

A. OFFSITE

The Company organization for station management and technical support shall be as shown on Figure 6.2.1.

B. FACILITY

The Facility organization shall be as shown on Figure 6.2.2 and:

1. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2.1.
2. At least one Licensed Operator shall be in the control room when fuel is in the reactor.
3. At least two Licensed Operators shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips.
4. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
5. ALL CORE ALTERATIONS performed while fuel is in the reactor vessel after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
6. The organization of the administration of the Pilgrim I Fire Prevention and Protection Program shall be as shown on Figure 6.2.3. A Fire Brigade of 5 members including the Fire Chief shall be maintained on site at all times. This excludes 3 members of the minimum shift crew necessary for safe shutdown and any personnel required for other essential functions during a fire emergency.

6.3 FACILITY STAFF QUALIFICATIONS

The qualifications with regard to educational and experience backgrounds of the facility staff at the time of appointment to the active position shall meet the requirements as described in the American National Standards Institute N18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants." In addition, the individual performing the function of Radiation Protection Manager shall meet or exceed the qualifications of Regulatory Guide 1.8, September, 1975.

6.4 TRAINING

- A. A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Operations Manager.
- B. A retraining program for the licensed operators shall be maintained under the direction of the Senior Nuclear Training Specialist and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55.
- C. A training program for the Fire Brigade shall be maintained under the direction of the Fire Protection and Prevention Officer and shall meet or exceed the requirements of Section 27 of the NFPA Code 1975. Training sessions will be held quarterly.

X

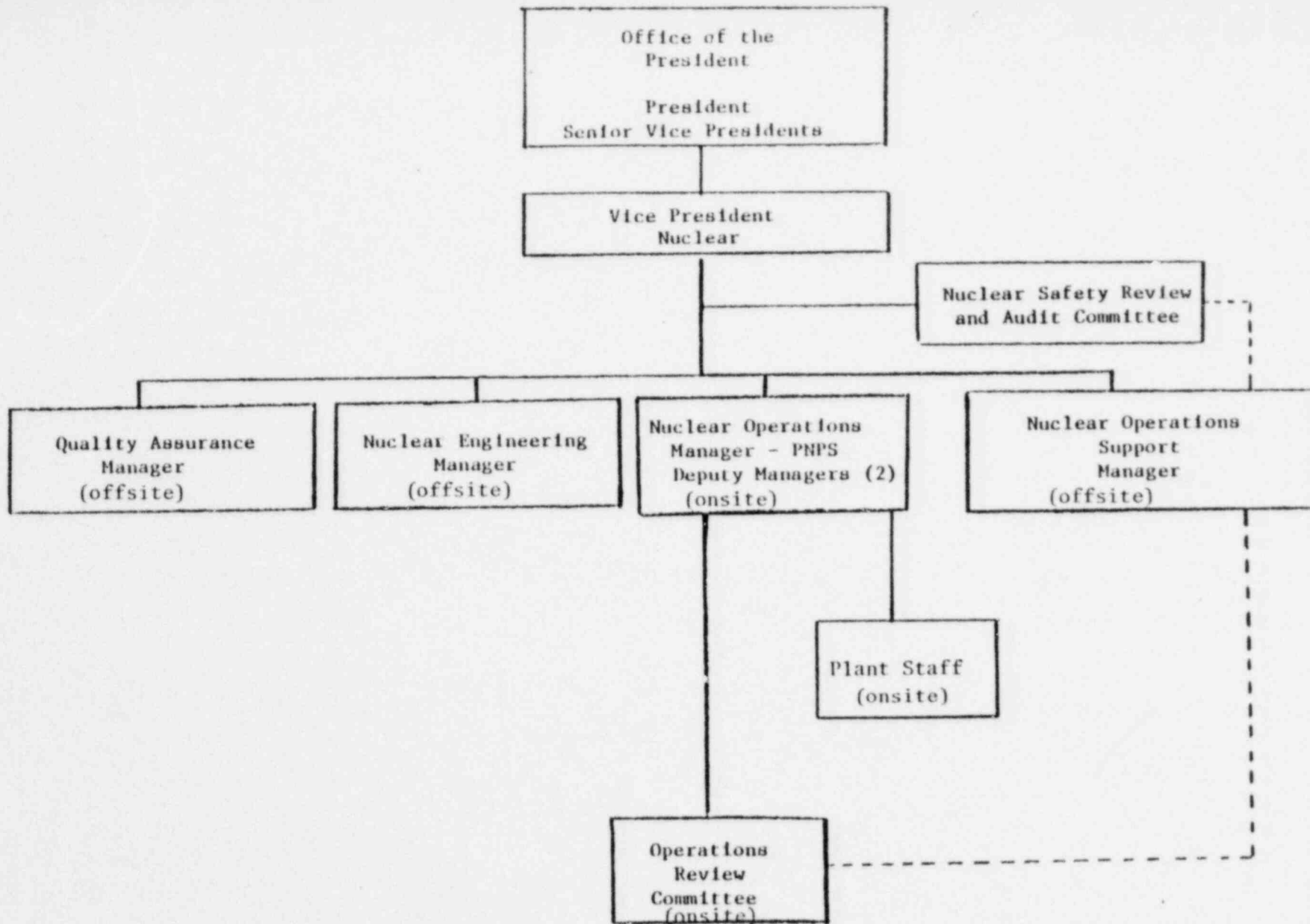
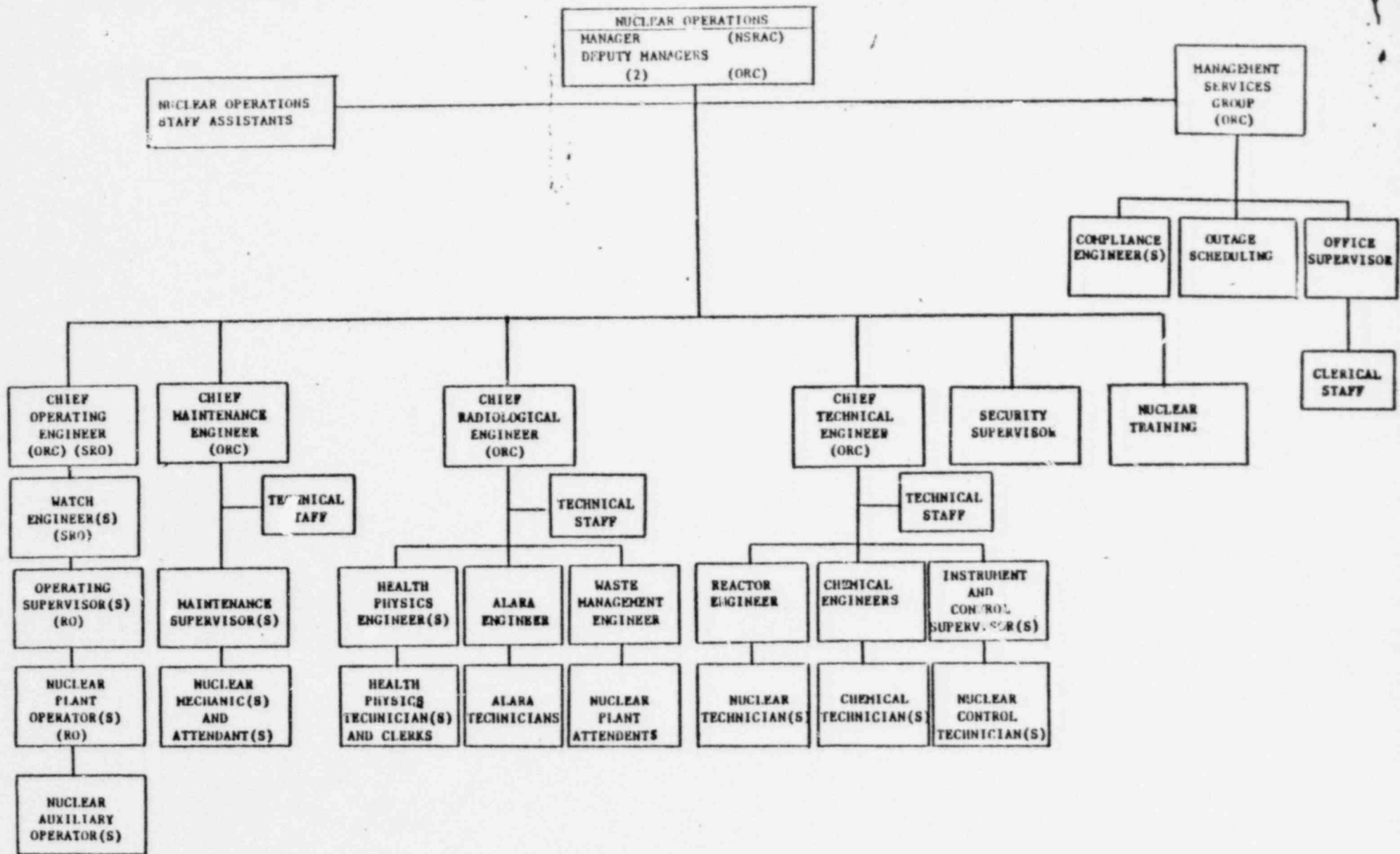


Figure 6.2.1

Boston Edison Company -- Organization for Operation

OF Pilgrim I

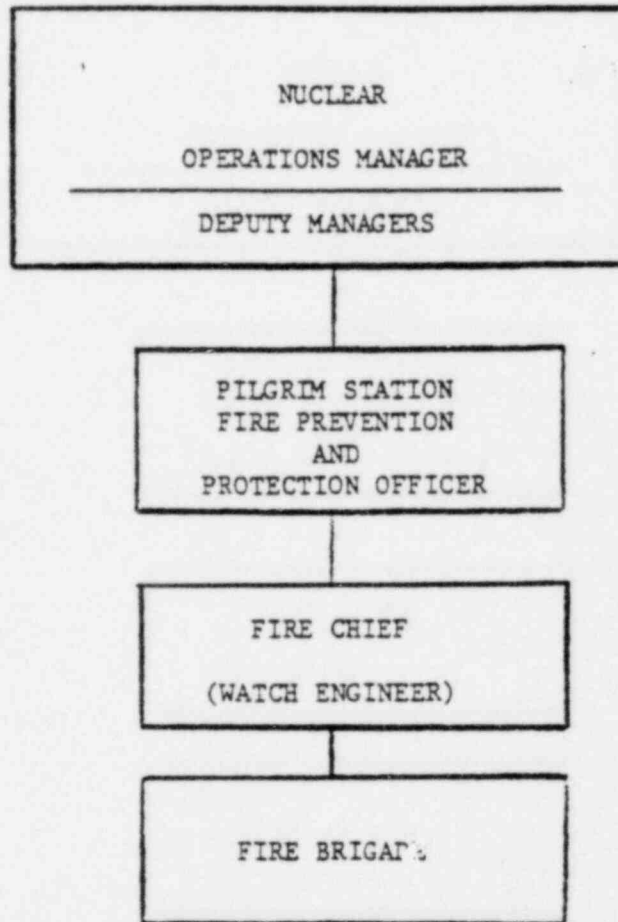


CODE: NSRAC - MEMBER OF NSRAC
 ORC - MEMBER OF ORC
 RO - NRC REACTOR OPERATOR LICENSE
 SRO - NRC SENIOR REACTOR OPERATOR LICENSE

PILGRIM 1 STATION ORGANIZATION
FIGURE 6.2.2

FIGURE 6.2.3

ADMINISTRATION OF THE PILGRIM NUCLEAR PLANT
FIRE PREVENTION AND PROTECTION PROGRAM



6.5 REVIEW AND AUDIT

A. OPERATIONS REVIEW COMMITTEE (ORC)

1. FUNCTION

The ORC shall function to advise the Nuclear Operations Manager on all matters related to nuclear safety.

2. COMPOSITION

The ORC shall be composed of the:

Chairman:	Deputy Nuclear Operations Managers
Member:	Management Services Group Leader
Member:	Chief Operating Engineer
Member:	Chief Technical Engineer
Member:	Chief Maintenance Engineer
Member:	Chief Radiological Engineer

3. ALTERNATES

Alternate members shall be appointed in writing by the ORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in an ORC quorum at any one time.

4. MEETING FREQUENCY

The ORC shall meet at least once per calendar month and as convened by the ORC Chairman.

5. QUORUM

A quorum of the ORC shall consist of the Chairman and three members including alternates.

6. RESPONSIBILITIES

The ORC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto that affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Nuclear Operations Manager and to the NSRAC Chairman.

- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and reports thereon as requested by the NSRAC Chairman.
- h. Review of the Station Security Plan and implementing procedures and changes to the plan and procedures.
- i. Review of the Emergency Plan and implementing procedures and changes to the plan and procedures.

7. AUTHORITY

The ORC shall:

- a. Inform the Nuclear Operations Manager in writing their approval or disapproval of items considered under 6.5.A.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.A.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Nuclear Operations Manager and the Nuclear Safety Review and Audit Committee of disagreement between the ORC members and the ORC Chairman. The Nuclear Operations Manager shall have responsibility for resolution for such disagreements pursuant to 6.1 above.

8. RECORDS

The ORC shall maintain written minutes of each meeting and copies shall be provided to the Nuclear Operations Manager and NSRAC Chairman.

B. NUCLEAR SAFETY REVIEW AND AUDIT COMMITTEE (NSRAC)

1. FUNCTION

The NSRAC shall function to provide independent review and audit of designated activities in the areas of:

- 1. nuclear power plant operations;
- 2. nuclear engineering;
- 3. chemistry and radiochemistry;
- 4. metallurgy;
- 5. instrumentation and control;
- 6. radiological safety;
- 7. mechanical and electrical engineering;
- 8. quality assurance practices
- 9. fire protection

2. COMPOSITION

The NSRAC Chairman and other members shall be appointed by the Vice President - Nuclear, or such other person as he shall designate. NSRAC members shall collectively possess experience and competence required to review and audit the designated areas noted in 6.5.3.1.

- b. Reports of reviews encompassed by Section 6.5.B.7 e, f, g and h above, shall be prepared, approved and forwarded to the Vice President-Nuclear, with a copy to the Nuclear Operations Manager within 21 days following the completion of the review.
- c. Audit reports encompassed by Section 6.5.B.8 above shall be forwarded to the Vice President-Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

The following actions shall be taken in the event of a reportable occurrence:

- A. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- B. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the ORC and submitted to the NSRAC Chairman and the Nuclear Operations Manager.

6.7 SAFETY LIMIT VIOLATION

The following actions shall be taken in the event a Safety Limit is violated:

- A. The provisions of 10 CFR 50.36(c) (1) (i) shall be complied with immediately.
- B. The Safety Limit Violation shall be reported to the Commission, the Nuclear Operations Manager and to the NSRAC Chairman immediately.
- C. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the ORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- D. The Safety Limit Violation Report shall be submitted to the Commission, the NSRAC Chairman and the Nuclear Operations Manager within 14 days of the violation.

6.8 PROCEDURES

- A. Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7 - 1972 and Appendix "A" of USNRC Regulatory Guide 1.33, except as provided in 6.3.B and 6.8.C below.

- B. Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the ORC and approved by the ORC Chairman prior to implementation and periodically as set forth in station procedures.
- C. Temporary changes to procedures of 6.8.A above may be made provided:
 - 1. The intent of the original procedure is not altered.
 - 2. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's license on the unit affected.
 - 3. The change is documented, reviewed subsequently by the ORC, and approved by the ORC Chairman within 7 days of implementation.
- D. Written procedures to implement the Fire Protection Program shall be established, implemented and maintained.

6.9 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

A. Routine Reports

- 1. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.