

Appendix A

NOTICE OF VIOLATION

Dairyland Power Cooperative

Docket No. 50-409

Based on the inspection conducted during August, 1980, it appears that certain of your activities were in noncompliance with NRC requirements, as noted below. Items 1, 2 and 3 are infractions, item 4 is a deficiency.

1. Section 50.59 of the Code of Federal Regulations requires that, prior to making changes to safety related systems, the license holder must perform a safety evaluation to assure that the proposed change does not constitute an unreviewed safety question.

Technical Specifications (TS) 3.5.1.6 and 3.5.1.7 require the Operations Review Committee (ORC) to review all proposed changes or modifications to plant systems or equipment that affects nuclear safety and recommend to the Plant Superintendent in writing approval or disapproval of the proposed change or modification. This review shall include a determination in writing as to whether or not the change or modification constitutes an unreviewed safety question.

Contrary to the above, a change discovered by the resident inspectors on July 23, 1980, was made to the Shutdown Condenser System sometime between September 1971 and July 1975 without the performance of a safety evaluation or a review by the ORC. Additionally, another facility change was made on July 23, 1980 without the performance of a safety evaluation or review by the ORC when the above unapproved modification was removed.

2. Technical Specification 2.1.2.5(1) requires that any piping system that is open to containment building atmosphere and outside atmosphere shall be provided with two valves in series.

Contrary to the above, an unauthorized modification was made to the shutdown condenser shell side vent, sometime between September 1971 and July 1975, which resulted in a piping system that did not meet the required two valve criteria, in that only one normally shut valve was provided.

3. Technical Specification 4.2.1.1 requires that containment integrity to be maintained whenever the reactor is operating at power.

Contrary to the above, on July 23, 1980, with the reactor at 85% power, containment integrity was violated for approximately one minute while an unauthorized modification was removed from the shell side of the shutdown condenser and replaced with a pipe cap.

4. 10 CFR Section 50.72 requires, in part, licensees of nuclear power reactors licensed under section 50.21 or section 50.32 to notify the NRC operations center as soon as possible and in all cases within one hour by telephone of any event resulting in automatic actuation of the reactor protection system.

Contrary to the above requirement, at 2:44 p.m. on August 22, 1980, the plant experienced a reactor trip, during startup at low power, due to a loss of feed water to the Forced Circulating Pump scale. This event was not reported within the one hour time limit stated above.