

YERA



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

September 26, 1980

Docket No. 50-155

17 AND
DISTRIBUTION SERVICES
BRANCH
NOV 6 PM 3 05
COMMUNICATIONS UNIT

Mr. David P. Hoffman
Nuclear Licensing Administrator
Consumers Power Company
212 West Michigan Avenue
Jackson, Michigan 49201

Dear Mr. Hoffman:

By letter dated November 16, 1979, we sent you the list of topics which would not be reviewed by your plant in the Systematic Evaluation Program (SEP). These topics were deleted from the SEP review because they were not applicable to your plant or they were being reviewed as part of an ongoing generic issue outside the SEP. The following topics should be added to the November 16 listing. Removal of these additional topics from the SEP review results from 1) related generic NRC activities and 2) non-applicability of topics to certain facilities. The following is a list of such topics and indicates their disposition:

<u>Topic</u>	<u>Categorization</u>	<u>Disposition</u>
II-2.B	Generic	Compliance with Appendix E is being evaluated under the Emergency Preparedness Program Office (EPPPO) effort. Compliance with Appendix I is being evaluated as part of the NRC Appendix I review effort.
II-2.D	Generic	Compliance is being evaluated under the EPPPO effort.
III-8.B	Deleted	The review published as NUREG-0479, "Report on BWR Control Rod Drive Failures" fulfills the intent of this topic.
VI-8	Generic	Will be reviewed by NRC as part of TMI Task Action Plan (NUREG-0660) as specified in May 7, 1980 letter to all operating reactor licensees.

Those parts of Topic II-1.B "Population Distribution" relating to emergency plans will be evaluated by EPPPO. In addition, the original DBE review list indicated that DBE group VI events "Uncontrolled Rod Assembly Withdrawal at Power and "Uncontrolled Rod Assembly Withdrawal - Low Power Startup" are a

8011180019


P

September 26, 1980

part of Topic XV-13. However, the topic of reference should be Topic XV-8, "Control Rod Misoperation" and accordingly the references for Topic XV-8 should include Standard Review Plans 15.4.1 and 15.4.2. Also, the November 16, 1979, letter states that DBE Group X, "Inadvertent Operation of ECCS or CVCS Malfunction that Causes an Increase in Reactor Coolant Inventory" should be deleted as it applies only to PWR's. However, Group X will be included in the SEP review since inadvertent operation of ECCS should be evaluated for BWR's.

Enclosure 1 to this letter includes a revised listing of the DBE Accident and Transient Groups applicable to BWR's. Of note should be the inclusion of Group XI, "Inadvertent Loading and Operation of a Fuel Assembly in an Improper Position", revision to Group III to include "Radiological Consequences Of a Main Steam Line Failure Outside Containment", and the Inclusion of Fuel Handling Accidents in Group VIII.

Sincerely,


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

cc

Mr. Paul A. Perry, Secretary
Consumers Power Company
212 West Michigan Avenue
Jackson, Michigan 49201

Judd L. Bacon, Esquire
Consumers Power Company
212 West Michigan Avenue
Jackson, Michigan 49201

Joseph Gallo, Esquire
Isham, Lincoln & Beale
1120 Connecticut Avenue
Room 325
Washington, D. C. 20036

Peter W. Steketee, Esquire
505 Peoples Building
Grand Rapids, Michigan 49503

Sheldon, Harmon and Weiss
1725 I Street, N. W.
Suite 506
Washington, D. C. 20006

Mr. John O'Neill, II
Route 2, Box 44
Maple City, Michigan 49664

Charlevoix Public Library
107 Clinton Street
Charlevoix, Michigan

Chairman
County Board of Supervisors
Charlevoix County
Charlevoix, Michigan 49720

Office of the Governor (2)
Room 1 - Capitol Building
Lansing, Michigan 48913

Director, Technical Assessment
Division
Office of Radiation Programs
(AW-459)
U. S. Environmental Protection
Agency
Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection
Agency
Federal Activities Branch
Region V Office
ATTN: EIS COORDINATOR
230 South Dearborn Street
Chicago, Illinois 60604

Herbert Grossman, Esq., Chairman
Atomic Safety and Licensing Board
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dr. Oscar H. Paris
Atomic Safety and Licensing Board
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. Frederick J. Shon
Atomic Safety and Licensing Board
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Big Rock Point Nuclear Power Plant
ATTN: Mr. C. J. Hartman
Plant Superintendent
Charlevoix, Michigan 49720

Christa-Maria
Route 2, Box 108C
Charlevoix, Michigan 49720

William J. Scanlon, Esquire
2034 Pauline Boulevard
Ann Arbor, Michigan 48103

Resident Inspector
Big Rock Point Plant
c/o U.S. NRC
RR #3, Box 600
Charlevoix, Michigan 49720

BWR DBE TOPIC LISTAccident and Transient Groups

	<u>RELATED TOPIC</u>	<u>DESIGN BASIS EVENT</u>	<u>SRP</u>
<u>Group I</u> (BWR)	XV-1	Decrease in feedwater temperature	15.1.1
	XV-1	Increase in feedwater flow	15.1.2
	XV-1	Increase in steam flow	15.1.3
	XV-9	Start-up of inactive loop and flow controller malfunction	15.4.4, 5
	XV-3	Inadvertent closure of main steam isolation valve	15.2.4
<u>Group II</u>	XV-3	Loss of external load	15.2.1
	XV-3	Turbine trip	15.2.2
	XV-3	Loss of condenser vacuum	15.2.3
	XV-3	Steam pressure regulatory failure	15.2.5
	XV-5	Loss of feedwater flow	15.2.7
<u>Group III</u>	XV-18	Radiological Consequences of a Main Steam Line Failure Outside Containment	15.6.4
<u>Group IV</u>	XV-4	Loss of AC power to station auxiliaries	15.2.6
	XV-24	Loss of all AC power	-
<u>Group V</u>	XV-7	Loss of forced coolant flow	15.3.1
	XV-7	Primary pump rotor seizure	15.3.3
	XV-7	Primary pump shaft break	15.3.4
<u>Group VI</u>	XV-8	Uncontrolled rod assembly withdrawal at power	15.4.2
	XV-8	Uncontrolled rod assembly withdrawal low power start-up	15.4.1
	XV-8	Control rod misoperation	15.4.3
	XV-13	Spectrum of rod drop accidents	15.4.9
<u>Group VII</u>	XV-19	Spectrum of loss of coolant accidents	15.6.5
<u>Group VIII</u>	XV-21	Drop of cask or heavy equipment	15.7.5
	XV-20	Radiological Consequences of Fuel Damaging Accidents (inside and outside containment)	15.7.4
<u>Group IX</u>	XV-15	Inadvertent opening of BWR safety/ relief valve	15.6.1
<u>Group X</u>	XV-14	Inadvertent operation of ECCS	15.5.1
<u>Group XI</u>	XV-11	Inadvertent loading and operation of a fuel assembly in an improper position	15.4.7
<u>Group XII</u>		Not applicable to BWRs	

BWR DBE LIST (continued)

<u>GROUP</u>	<u>RELATED SEP TOPIC</u>	<u>DESIGN BASIS EVENT</u>	<u>SRP SECTION</u>
VI	XV-8	Uncontrolled rod assembly withdrawal at power	15.4.2
	XV-8	Uncontrolled rod assembly withdrawal low power start-up	15.4.1
	XV-8	Control rod misoperation	15.4.3
	XV-13	Spectrum of rod drop accidents	15.4.9
VII	XV-19	Spectrum of rod drop accidents	15.6.5
VIII	XV-20	Radiological consequences of fuel damaging accidents (inside and outside containment)	15.7.4
	XV-21	Drop of cask or heavy equipment	15.7.5
IX	XV-15	Inadvertent opening of BSR safety/relief valve	15.6.1
X	XV-14	Inadvertent operation of ECCS	15.5.1
XI	XV-11	Inadvertent loading and operation of a fuel assembly in an improper position	15.4.7
XII	-	Not applicable to BWR's	

PWR DBE LIST

<u>GROUP</u>	<u>RELATED SEP TOPIC</u>	<u>DESIGN BASIS EVENT</u>	<u>SRP SECTION</u>
I	XV-1	Decrease in feedwater temperature	15.1.1
	XV-1	Increase in feedwater flow	15.1.2
	XV-1	Increase in steam flow	15.1.3
	XV-1	Inadvertent opening of steam generator relief/safety valve	15.1.4
	XV-9	Start-up of inactive loop	15.4.4
	XV-10	System malfunction causing a decrease in boron concentration	15.4.6
II	XV-3	Loss of external load	15.2.1
	XV-3	Turbine trip	15.2.2
	XV-3	Loss of condenser vacuum	15.2.3
	XV-3	Steam pressure regulatory failure	15.2.5
	XV-5	Loss of feedwater flow	15.2.7
	XV-6	Feedwater system pipe break	15.2.8