



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20565

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated September 15, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

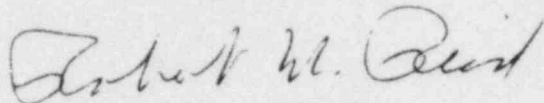
Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 74, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Robert W. Reid".

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 28, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Page 85 was not changed; it is included for convenience only.

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TABLE 3.2.F

SURVEILLANCE INSTRUMENTATION

Minimum No. of Operable Instrument Channels	Instrument	Type Indication and Range	Action
2	Reactor Water Level	Recorder 0-60" Indicator 0-60"	(6) (7)
2	Reactor Pressure	Recorder 0-1500 psig Indicator 0-1200 psig	(1) (2) (3)
2	Drywell Pressure	Recorder 0-70 psig	(1) (2) (3)
2	Drywell Temperature	Recorder 0-400 ^o F Indicator 0-400 ^o F	(1) (2) (3)
2	Suppression Chamber Water Temperature	Recorder 0-600 ^o F Indicator 0-400 ^o F	(1) (2) (3)
2	Suppression Chamber Water Level	Recorder 0-2 ft. Indicator 0-2 ft.	(1) (5)
1	Control Rod Position	28 Volt Indicating Lights)))	(1) (2) (3) (4)
1	Neutron Monitoring	SRM, IRM, LPRM) 0-100%)	
1	Safety-Relief Valve Position Indication	Acoustic or thermocouple	(8)

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NOTES FOR TABLE 3.2.F

- 1) From and after the date that one of these parameters is reduced to one indication, continued operation is permissible during the succeeding thirty days unless such instrumentation is sooner made operable.
- 2) From and after the date that one of these parameters is not indicated in the control room, continued operation is permissible during the succeeding seven days unless such instrumentation is sooner made operable.
- 3) If the requirements of notes (1) and (2) cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold condition within 24 hours.
- 4) These surveillance instruments are considered to be redundant to each other.
- 5) In the event that all indications of this parameter are disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a Hot Shutdown condition in six (6) hours and a Cold Shutdown condition in the following eighteen (18) hours.
- 6) With the number of operable channels less than the minimum number of instrumentation channels shown in Table 3.2.F, either restore the inoperable channel to an operable status within 7 days, or be in at least hot shutdown within the next 12 hours.
- 7) If this parameter is not indicated in the control room, either restore at least one inoperable channel to operable status within 48 hours or be in at least hot shutdown within the next 12 hours.
- 8) If this parameter is not indicated in the control room, either restore at least one channel to operable status within thirty days or be in at least hot shutdown within the next 12 hours.

TABLE 4.2.E

MINIMUM TEST AND CALIBRATION FREQUENCY FOR DRYWELL LEAK DETECTION

<u>Instrument Channel</u>	<u>Instrument Functional Test</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) Equipment Drain Sump Flow Integrator	(1)	Once/3 months	Once/day
2) Floor Drain Sump Flow Integrator	(1)	Once/3 months	Once/day
3) Air Sampling System	(1)	Once/3 months	Once/day

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TABLE 4.2.F

UNIT 2

<u>Instrument Channel</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) Reactor Level	Once/operating cycle	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Temperature	Once/6 months	Once Each Shift
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring (APRN)	Twice Per Week	Once Each Shift
9) Safety/Relief Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
10) Safety/Relief Valve Position Indicator (thermocouple)	NA*	Once/month
11) Safety Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
12) Safety Valve Position Indicator (thermocouple)	NA*	Once/month

* Perform instrument functional check once per operating cycle

TABLE 4.2.8

UNIT 2

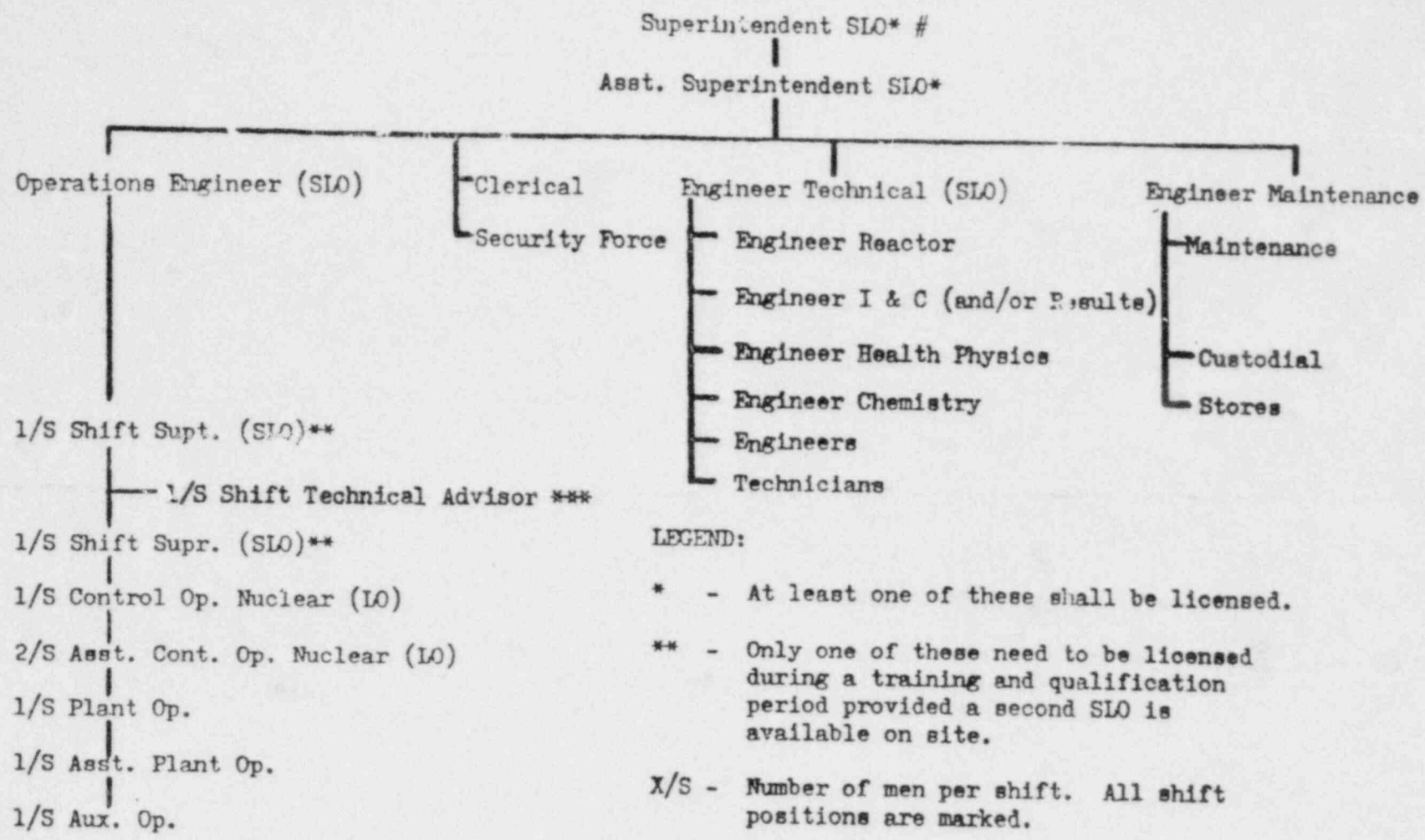
MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

<u>Instrument Channel</u>	<u>Calibration Frequency</u>	<u>Instrument Check</u>
1) Reactor Level	Once/operating cycle	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Temperature	Once/6 months	Once Each Shift
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring (APRN)	Twice Per Week	Once Each Shift
9) Safety/Relief Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
10) Safety/Relief Valve Position Indicator (thermocouple)	NA*	Once/month
11) Safety Valve Position Indicator (acoustics)	Once/operating cycle	Once/month
12) Safety Valve Position Indicator (thermocouple)	NA*	Once/month

* Perform instrument functional check once per operating cycle

Amendment No. 10, 39, 61, 74

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LEGEND:

- * - At least one of these shall be licensed.
- ** - Only one of these need to be licensed during a training and qualification period provided a second SLO is available on site.
- X/S - Number of men per shift. All shift positions are marked.
- LO - NRC Licensed Operator
- SLO - NRC Licensed Senior Operator
- # - Responsible for onsite fire protection activities
- *** - Not applicable with plant in the shutdown or refuel mode.

ORGANIZATION FOR CONDUCT OF PLANT OPERATIONS

Figure 6.2-2

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6.3 Facility Staff Qualifications

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N13.1-1971 for comparable positions, except for (1) the Engineer-Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975 and (2) the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 Training

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Station Superintendent and shall meet the requirements of Section 5.5 of ANSI N13.1-1971 and 10 CFR 55, Appendix A.
- 6.4.2 A training program for the Fire Brigade shall be conducted such that Fire Brigade members complete an instruction program within a two year period. Regularly planned meetings will be held every 3 months.

6.5 Review and Audit

6.5.1 Plant Operation Review Committee (PORC)

Function

- 6.5.1.1 The Plant Operation Review Committee shall function to advise the Station Superintendent on all matters related to nuclear safety.

Composition

- 6.5.1.2 The Plant Operation Review Committee shall be composed of the:

Station Superintendent-Chairman
Station Assistant Superintendent
Engineer - Technical
Engineer - Maintenance
Engineer - Operations
Engineer - Results
Engineer - Reactor
Engineer - Instrument & Control
Engineer - Health Physics
Engineer - Chemistry
Shift Superintendent

Alternates

- 6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.

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6.14 Integrity of Systems Outside Containment

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

- 1) Provisions establishing preventive maintenance and periodic visual inspection requirements, and
- 2) System leak test requirements, to the extent permitted by system design and radiological conditions, for each system at a frequency not to exceed refueling cycle intervals. The systems subject to this testing are (1) Residual Heat Removal, (2) Core Spray, (3) Reactor Water Cleanup, (4) HPCI, and (5) RCIC.

6.15 Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas* under accident conditions. This program shall include the following:

1. Training of personnel,
2. Procedures for monitoring, and
3. Provisions for maintenance of sampling and analysis equipment.

* Areas requiring personnel access for establishing hot shutdown condition.



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PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. DPR-56

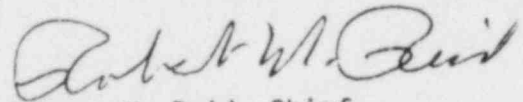
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 - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated September 15, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "Robert W. Reid".

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 28, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Page 85 was not changed; it is included for convenience only.

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1	Neutron Monitoring	SRM, IRM, LPRM) 0-100%)	
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TABLE 4.2.F

UNIT 3

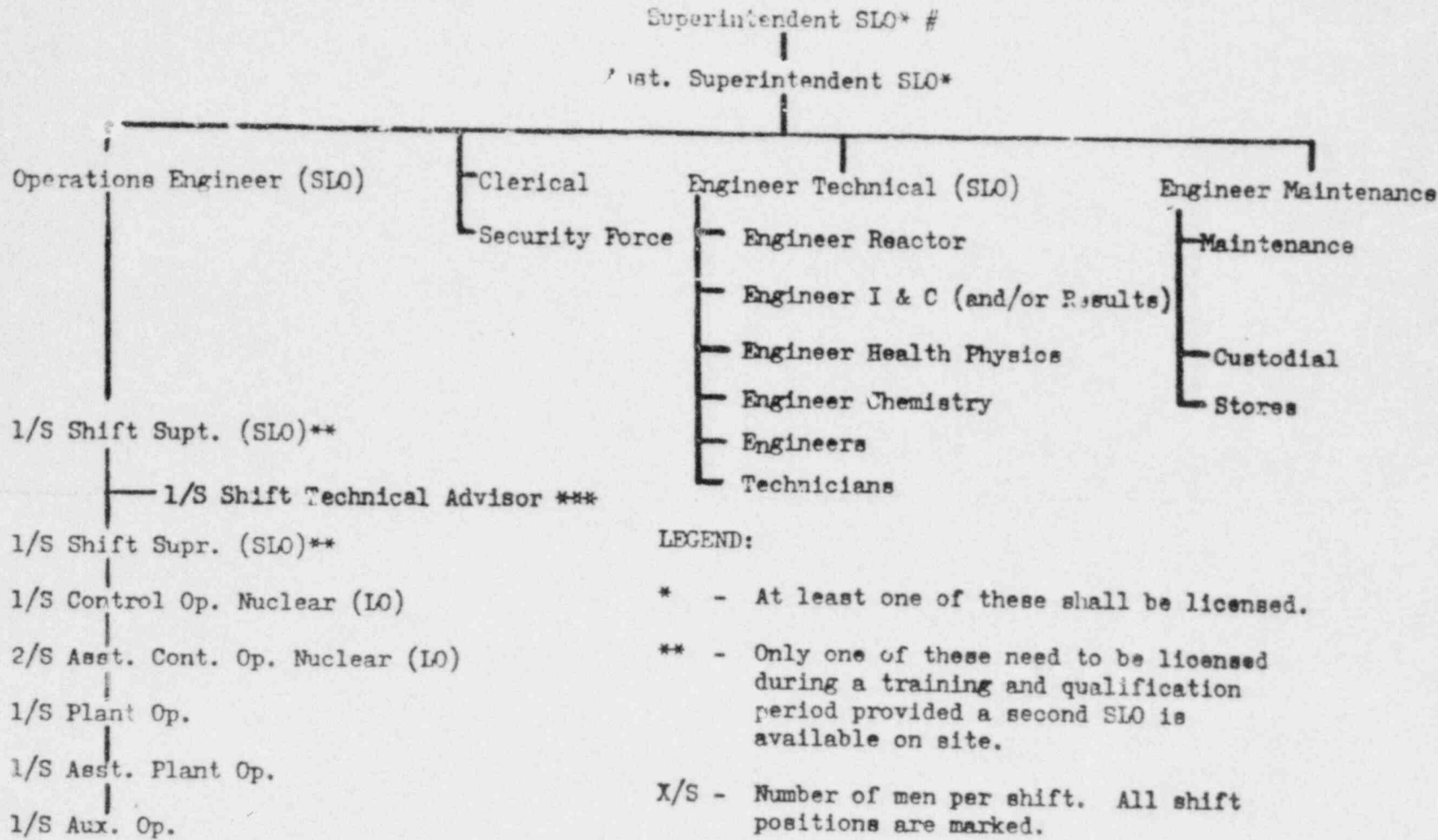
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12) Safety Valve Position Indicator (thermocouple)	NA***	Once/month

* Deleted when modifications authorized by Amendment No. 67 are completed.

** Effective when modifications authorized by Amendment No. 67 are completed.

*** Perform instrument functional check once per operating cycle.



LEGEND:

- * - At least one of these shall be licensed.
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- X/S - Number of men per shift. All shift positions are marked.
- LO - NRC Licensed Operator
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ORGANIZATION FOR CONDUCT OF PLANT OPERATIONS

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Function

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- Engineer - Maintenance
- Engineer - Operations
- Engineer - Results
- Engineer - Reactor
- Engineer - Instrument & Control
- Engineer - Health Physics
- Engineer - Chemistry
- Shift Superintendent

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