

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No: 50-282/80-14; 50-306/80-15

Docket No: 50-282; 50-306

License No: DPR-42; DPR-60

Licensee: Northern States Power Company
414 Nicollet Mall
Minneapolis, MN. 55401

Facility Name: Prairie Island Nuclear Generating Plant

Inspection At: Prairie Island Site, Red Wing, MN. 55066

Inspection Conducted: August 1-31, 1980

Inspectors:

C.D. Feierabend
C.D. Feierabend

9/3/80

B.L. Burgess
for B.L. Burgess

9/3/80

Approved By: *W.S. Little*
for W.S. Little, Chief
Reactor Projects Section 2

9/19/80

Inspection Summary

Inspection on August 1-31, 1980 (Report No. 50-282/80-14; 50-306/80-15)

Areas Inspected: Routine resident inspection of plant operation, maintenance, surveillance, security, radiation protection, organization, training and followup on I.E. Bulletins. The inspection involved 89 inspection hours onsite by two NRC inspectors. This included 15 hours of offshift inspection.

Results: No items of noncompliance were identified.

DETAILS

1. Personnel Contacted

F. Tierney, Plant Manager
J. Brokaw, Plant Superintendent, Operations and Maintenance
E. Watzl, Plant Superintendent, Plant Engineering and Radiation Protection
A. Hunstad, Staff Engineer
R. Lindsey, Superintendent, Operations
J. Nelson, Superintendent, Maintenance
J. Hoffman, Superintendent, Technical Engineering
D. Mendele, Superintendent, Operations Engineering
D. Schuelke, Superintendent, Radiation Protection
A. Smith, Senior Scheduling Engineer
M. Klee, Nuclear Engineer
R. Pearson, Engineer
D. Brown, Engineer
K. Albrecht, Superintendent, Quality Assurance Engineering
D. Cragoe, Shift Supervisor
G. Edon, Shift Supervisor
M. Balk, Shift Supervisor
D. Walker, Shift Supervisor
P. Valtakis, Shift Supervisor

2. Operational Safety Verification

a. General

Unit 2 operated routinely throughout the month. Unit 1 was shut down on August 31 to begin an extended refueling and maintenance outage.

Plant visitors during the month included members of the Atomic Safety and Licensing Board Panel on August 5, and the NRR Project Manager for Prairie Island on August 7.

b. Control Room Observations

The inspector observed control room operations, reviewed applicable logs, conducted discussions with control room operators and observed shift turnovers. The inspector verified the operability of selected emergency systems, reviewed equipment control records, and verified proper return to service of affected components.

c. Tours

Tours of the auxiliary, turbine and shield buildings and external areas were conducted to observe plant equipment conditions, including potential fire hazards, and to verify that maintenance requests had been initiated for equipment in need of maintenance. By observation and direct interview, the inspector verified that security procedures were being implemented in accordance with the plant security plan.

The inspectors observed plant housekeeping/cleanliness conditions, and verified implementation of radiation protection controls. This included observing decontamination of the irradiated specimen cask prior to release for shipment.

d. Independent Verification

The inspectors performed walkdowns of the accessible portions of the cooling water and auxiliary feedwater systems. This included confirmation of selected portions of the licensee's procedures, check-lists, and plant drawings, inspection of condition of components and piping supports, inspection of breakers and cabinets, verification of selected instruments for proper valving, equipment control locks or tags and comparison of remote and local indication.

The inspector also performed independent verification of the D1 diesel generator and of the No.11 auxiliary feedwater pump after return to service following completion of maintenance and surveillance tests.

e. Verification of Plant Conditions

The inspectors independently verified the status of plant and equipment conditions to confirm fulfillment of Technical Specifications (TS) limiting condition for operation (LCO) requirements. The following LCO's were verified:

TS 3.3.C	Component Cooling Water System
TS 3.3.D	Cooling Water System
TS 3.4	Steam and Power Conversion System
TS 3.7	Auxiliary Electrical System
TS 3.10.B2	Power Distribution Limits
TS 3.10.D	Rod Insertion Limits

3. Organization and Administration

The licensee has transferred responsibility for all training from the plants to the General Office under direction of Nuclear Safety Technical Services. This will require minor change to the Technical Specifications which will be included in the next routine Technical Specification change request.

4. Maintenance

- a. Review of Work Request (WR's) and Work Request Authorizations (WRA's)

The inspector selected and reviewed several WR's and WRA's to determine the status of safety related systems, to verify that proper priorities were given and to verify that design changes were initiated where appropriate.

b. Observations

The inspector observed portions of safety related maintenance activities to determine that the activities did not violate limiting conditions for operation (LCO's), that administrative approvals and equipment control tags were completed prior to initiating the work, that approved procedures were used (or activity was within the "skills of the trade"), and that the procedures used were adequate to control the activity.

The inspectors witnessed maintenance activities during replacement of the oil cooler on D2 diesel generator and during annual preventive maintenance of D1 diesel generator.

5. Surveillance

The inspectors witnessed portions of surveillance testing of safety related systems and components. Witnessing included verifying that the tests were scheduled and performed within Technical Specification requirements, observing that procedures were being followed, that LCO's were not violated and that system restoration was completed.

Tests witnessed included portions of tests conducted on the D1 and D2 diesel generators, the No.12 auxiliary feedwater pump 48 hour endurance test and the main steam safety valves.

6. Training

The inspectors monitored a retraining lecture/discussion conducted for Technical staff senior license holders. Subject of the training was the control rod drive system.

7. I.E. Bulletins

The inspector reviewed the licensee's written response to the following bulletins, verified that the response included the required information and confirmed that the information in the response was accurate.

- a. I.E.Bulletin 80-04 Analysis of a PWR Main Steam Line Break with Continued Feedwater Addition

Licensee analysis indicates that no corrective action is required. Additional plant specific analysis is expected to be completed later this year. (Open)

- b. I.E. Bulletin 80-08 Examination of Containment Liner Penetration Welds

Licensee response was not complete. The licensee has supplemented the response, which is being reviewed by I.E. construction specialists. (Open)

- c. I.E. Bulletin 80-09 Hydramotor Actuator Deficiencies

There are no actuators of this type installed in safety related systems. (Closed)

- d. I.E. Bulletin 80-18 Minimum Flow Through Centrifugal Charging Pumps

Not applicable. (Closed)

8. Exit Interviews

The inspectors conducted interim interviews with Messrs. Watzl and Tierney and met with Mr. Tierney at the conclusion of the inspection. The inspector discussed the scope and results of the inspection. No items of noncompliance or deviations were identified.