

NOTICE
AS OF MAY 17 1976
REGION I HAS NOT OBTAINED PROPRIETARY
CLEARANCE IN ACCORDANCE WITH 10 CFR 27.41

U. S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
REGION I

IE Inspection Report No: 50-29/76-11 Docket No: 50-29

Licensee: Yankee Atomic Electric Company License No: DPR-3
20 Turnpike Road Priority: --
Westborough, Massachusetts 01581 Category: C

Location: Rowe, Massachusetts Safeguards Group: --

Type of Licensee: PWR (W) 600 MWt

Type of Inspection: Routine, Unannounced

Dates of Inspection: April 12-16, 1976

Dates of Previous Inspection: March 29-31, 1976

Reporting Inspector: *D. R. Haverkamp* 5/14/76
D. R. Haverkamp, Reactor Inspector DATE

Accompanying Inspectors: *A. B. Davis* 5/14/76
A. B. Davis, Section Chief DATE
(part time)

DATE

DATE

Other Accompanying Personnel: None DATE

viewed By: *A. B. Davis* 5/14/76
A. B. Davis, Section Chief, Reactor Projects DATE
Section No. 1, Reactor Operations and Nuclear
Support Branch

8011060 857

POOR ORIGINAL

SUMMARY OF FINDINGS

Enforcement Action

Items of Noncompliance

Infractions

- A. Contrary to Technical Specification D.1 requirements to conduct plant operations in accordance with written procedures, the power range neutron flux level channel amplifiers were adjusted without adherence to the requirements of procedure OP-4201. (Detail 4.c.(2))
- B. Contrary to Technical Specification D.1 requirement for modifying written operating procedures, changes were improperly made to procedure OP-2163. (Detail 7.c.(1))

Licensee Action on Previously Identified Enforcement Items

Item of Noncompliance - Region I Inspection Report 50-29, 76-06, Detail 6.b

The licensee's corrective actions with respect to the above item of noncompliance was reviewed and found to be acceptable and complete. (Detail 8)

Design Changes

Not inspected.

Unusual Occurrences

None identified.

Other Significant Findings

A. Current Findings

1. Acceptable Areas

- a. Shift logs and Operating Records. (Detail 3)
- b. Plant Tour. (Detail 4, except 4.c.(2))
- c. Reportable Occurrences. (Detail 5, except 5.b and 5.c)
- d. Nonroutine Event Review. (Detail 6)
- e. Safety Limits, Limiting Safety System Settings and Limiting Conditions for Operation. (Detail 7, except 7.c.(1))

2. New Unresolved Items

- a. Reportable Occurrence 76-01. (Detail 5.b)
- b. Reportable Occurrence 76-02. (Detail 5.c)

3. Licensee Identified Items of Noncompliance and Deviations

None identified.

B. Status of Previous Unresolved Items

- 1. The following items were reviewed and are considered resolved:
 - a. Procedure Changes. (Detail 10.a)
 - b. Switching Log. (Detail 11)
 - c. Fire Protection Training. (Detail 12)
 - d. Identification of Safety Related Instrumentation and Controls. (Detail 13)
 - e. Reported Radiochemistry Data. (Detail 14)

2. The following items were reviewed and remain unresolved:
 - a. Reportable Occurrences 75-07, 76-01 and 76-02.
(Detail 9.a)
 - b. Reportable Occurrence 75-12. (Detail 9.b)
 - c. Reportable Occurrence 75-17. (Detail 9.c)
 - d. Procedure Changes. (Detail 10.b)

Management Interviews

A. Entrance Interview

A pre-inspection interview was conducted on site at the beginning of the inspection on April 12 with the following licensee personnel in attendance:

Mr. H. A. Autio, Plant Superintendent
Mr. J. L. Staub, Technical Assistant to Plant Superintendent

The licensee identified no operational problems that related to plant safety or radiological health since the last inspection.

The inspector identified the scope and objectives of the inspection and scheduled an exit meeting for April 16.

B. Exit Interview

An exit interview was conducted on site at the conclusion of the inspection on April 16 with the following licensee personnel in attendance:

Mr. H. A. Autio, Plant Superintendent
Mr. W. D. Billings, Chemistry and Health Physics Supervisor
Mr. R. L. Boutwell, Technical Assistant
Mr. T. D. Danek, Operations Supervisor
Mr. M. W. Ebert, Reactor Supervisor
Mr. P. E. Laird, Maintenance Supervisor
Mr. I. R. Seybold, Plant Health Physicist

Mr. J. H. Shippee, Instrumentation and Control Supervisor
Mr. J. L. Staub, Technical Assistant to Plant Superintendent
Mr. N. N. St. Laurent, Assistant Plant Superintendent
Mr. D. B. Vassar, Assistant Operations Supervisor

The scope and objectives of the inspection were discussed and the inspection findings were presented as they appear in this report.

DETAILS

1. Persons Contacted

H. A. Autio, Plant Superintendent
R. L. Berry, Technical Assistant
W. D. Billings, Chemistry and Health Physics Supervisor
R. L. Boutwell, Technical Assistant
C. J. Crosier, Tester
T. P. Danek, Operations Supervisor
M. W. Ebert, Reactor Supervisor
J. S. Gedutis, Technical Assistant
J. M. Grillo, Control Room Operator
R. A. Herzog, Shift Supervisor
B. L. Kirk, Shift Supervisor
P. E. Laird, Maintenance Supervisor
L. Pritt, Auxiliary Operator
I. R. Seybold, Plant Health Physicist
J. H. Shippee, Instrument and Control Supervisor
J. L. Staub, Technical Assistant to Plant Superintendent
N. N. St. Laurent, Assistant Plant Superintendent
J. C. Trejo, Engineering Assistant

2. Inspection Purpose

The inspector stated that the purpose of the inspection was to:

- a. Review Plant Operations.
- b. Review Selected Nonroutine Event Reports and Reporting Responsibilities.
- c. Review Safety Limit, Limiting Safety System Settings and Limiting Conditions for Operation.
- d. Review Items of Noncompliance and Unresolved Items Identified in Previous Inspections.

The licensee acknowledged this information.

3. Shift Logs and Operating Records

- a. Shift logs and operating records were reviewed to verify that:
- (1) Control Room log sheet entries are filled out and initialed.
 - (2) Auxiliary log sheets are filled out and initiated.
 - (3) Shift Supervisor and Control Room log entries involving abnormal conditions provide sufficient detail to communicate equipment status, lockout status, correction and restoration.
 - (4) Log Book reviews are being conducted by the staff.
 - (5) Operating orders do not conflict with Technical Specifications requirements.
 - (6) Jumper (Bypass) log does not contain bypassing discrepancies with Technical Specification requirements.
 - (7) "Problem Identification Reports" confirm there are no violations of Technical Specifications reporting or LCO requirements.
- b. The review included discussions with licensee personnel and review of Technical Specifications and the following plant procedures, shift logs and operating records;
- (1) AP-0001 Plant Procedures, Rev. 4, dated June 13, 1975.
 - (2) AP-0004 Plant Information Reports, Rev. 2, dated August 8, 1975.
 - (3) AP-0018 Bypass of Safety Function and Jumper Control, Rev. 2, dated September 15, 1975.

- (4) AP-0021 Operating Memos, Rev. 1, dated December 20, 1974.
 - (5) AP-0219 Maintenance of Operations Department Logs, Rev. 1, dated June 15, 1974.
 - (6) AP-2006 Special Orders, Rev. 0, dated October 31, 1974.
 - (7) Shift Supervisor (Operations) Log, January 1, 1976 - March 31, 1976.
 - (8) Rowe Station Log 1, January 1, 1976 - March 31, 1976.
 - (9) Rowe Station Log 2, January 1, 1976 - March 31, 1976.
 - (10) Primary Plant Log Sheet, January 1, 1976 - March 31, 1976.
 - (11) Secondary Plant Log Sheet, January 1, 1976 - March 31, 1976.
 - (12) Operating Memos ZE-2, dated March 1, 1976; ZK-4 dated March 29, 1976; and ZR-25, dated March 16, 1976.
 - (13) Night Order Book, January 1, 1976 - March 31, 1976.
 - (14) Bypass of Safety Function and Jumper Control Log, January 1, 1976 - March 31, 1976.
 - (15) Bypass of Safety Function and Jumper Control Requests 76-1 through 76-50.
 - (16) Plant Information Reports 76-1 through 76-3.
 - (17) Minutes of Plant Operations Review Committee (PORC) Meetings 76-1 through 76-19.
- c. The inspector noted an improvement in the overall quality of operating logs and records since the last inspection of this area. No discrepancies were identified.

4. Plant Tour

- a. The inspector toured accessible areas of the plant, including the Control Room, Vapor Container, Primary Auxiliary Building, Safety Injection/Diesel Building and Turbine Building on April 13 and 15. The tours were conducted to verify that:
- (1) Monitoring instrumentation for pressurizer level, accumulator volume and nitrogen pressure, and boric acid mixing and storage tank level were recording system/component parameters as required.
 - (2) Radiation controls were properly established in the vapor container and primary auxiliary building.
 - (3) Plant housekeeping conditions were acceptable.
 - (4) No unusual fluid leaks or piping vibrations existed.
 - (5) Pressurizer safety valves and piping hydraulic snubber oil levels were satisfactory.
 - (6) Selected safety injection system valves were properly positioned.
 - (7) Selected equipment tags were properly authorized and logged.
 - (8) The control room operators were aware of the reasons for lighted annunciators and had taken action specified in applicable alarm procedures.
 - (9) Plant tours conducted by the Plant Superintendent and Shift Supervisor are consistent with administrative procedures.
 - (10) Control Room manning was in conformance with the Technical Specifications and 10 CFR 50.54(k).
- b. The tours included discussions with licensee personnel, observation of the above conditions and components, measurement of actual radiation levels at selected areas of the vapor container, and review of Technical Specifications and the following plant procedures and records:

- (1) AP-0017 Switching and Tagging Rules for Plant Equipment, Rev. 1, dated October 25, 1975.
 - (2) OP-4203 Weekly Valve Check, Rev. 5, dated December 5, 1975.
 - (3) OP-8100 Establishing and Posting Controlled Areas, Rev. 2, dated May 2, 1975.
 - (4) Switching Log, January 1, 1976 - March 31, 1976.
 - (5) Tagging Status Log, January 1, 1976 - March 31, 1976.
 - (6) Switching Orders 9113U, 1910U and 9581U.
 - (7) Radiation Work Permit No. 438.
- c. The inspector determined that the items reviewed and observed during the tours were acceptable, except as described below.
- (1) Some caution signs and posted radiation levels inside the vapor container differed from the existing measured radiation levels. The inconsistency was caused by posting the caution signs and annotating the radiation levels during a maintenance shutdown in January 1976. When power operation was resumed these signs were left in place with no subsequent updating to reflect the change in the actual radiation levels. During normal plant operation the vapor container is considered and controlled by the licensee as a high radiation area. These controls were reviewed by the inspector and found acceptable. However, the radiation signs inside the vapor container are misleading. The licensee stated that the signs would be removed, covered or otherwise corrected during the next routine entry on April 20. The inspector had no further questions in this area at this time.
 - (2) While conducting a tour of the Control Room, the inspector observed the Control Room Operator adjusting

the gain potentiometers for the power range neutron flux level channel amplifiers. The adjustments were made in a nonconservative direction such that subsequent operations could have resulted in exceeding the allowable fraction of full power, as defined in Technical Specification D.2.c.(1). OP-4201, Rev. 5, "Power Range Channel Calibration," specifies the manner in which power range neutron flux level channel amplifiers will be adjusted. The gain potentiometer adjustments were not made in conformance with the requirements of OP-4201. The improper performance of OP-4201 is an Item of Noncompliance, Infraction level.

Prior to the completion of the inspection, the licensee performed the following corrective actions to prevent recurrence of the item of noncompliance.

- (a) The Control Room operators were immediately verbally instructed to make adjustments to power range channels in accordance with OP-4201.
- (b) A memorandum dated April 15, 1976, "Setting of Power and Intermediate Range Channels," was issued by the Plant Superintendent to all Shift Supervisors and Control Room Operators. This memorandum provided revised instructions for setting the calibration level of the subject channels.
- (c) Revision 12 to AP-7104, "Core XII Operational Limits," was issued and approved on April 15, 1976. This revision incorporated the instructions of the above memorandum. The inspector noted an error that appeared in the revised procedure, and the licensee stated this item would be corrected by April 30, 1976.
- (d) A revision to OP-4201 was initiated to be consistent with the current instructions for determining power and intermediate range channel calibration levels. The licensee stated that the revised procedure would be issued by April 30, 1976.

The actions taken or intended to be taken to correct this Item of Noncompliance and to prevent its recurrence are acceptable. The licensee's completion of corrective action, including revisions to AP-7104 and OP-4201, will be reviewed during a subsequent inspection.

- (3) The tour of the vapor container was conducted in company with the Shift Supervisor, who was performing OP-4232, "Bi-weekly Inspection of Vapor Container." The Shift Supervisor identified a packing leak from no. 1 loop valve stem leakoff line and issued MR #76-131 to effect the necessary corrective maintenance. The shift supervisor's inspection was observed to be systematic and thorough. The inspector had no questions in this area.

5. Reportable Occurrences

- a. Reportable occurrences discussed below were reviewed to verify that:

- (1) Details were clearly reported to the NRC and Facility Management.
- (2) Corrective action described in the report was taken to prevent recurrence.
- (3) The occurrence was reviewed and evaluated as required by the Technical Specifications.
- (4) Safety Limits, Limiting Safety System Settings and Limiting Conditions for Operation were not exceeded.

These areas were satisfactory for the occurrences reviewed unless otherwise noted.

- b. Reportable Occurrence 76-01

Reference: Licensee letter to NRC Region I dated March 4, 1976.

This occurrence concerned leakage from cracked welds in a piping flange and tee in the no. 3 charging pump relief valve discharge line. There was no detectable increase in air activity in the area of the leakage. The cause of the weld cracks was attributed to vibration induced fatigue failure caused by continuous operation of the positive displacement charging pump. Charging was transferred to the no. 1 charging pump, and the flange and tee were replaced and leak tested satisfactorily. The licensee's action concerning this occurrence were documented in the following records.

- (1) Maintenance Request #76-125.
- (2) Job Order #76-28.
- (3) OP-5100, Rev. 3, "Valve, Fitting or Pipe Section Replacement and/or Repair," completed February 4, 1976.
- (4) PORC Minutes, Meeting 76-11.
- (5) Breathing zone air sample results for Cubicle #3 Charging Pump dated February 4, 1976.
- (6) YAEC Surface Contamination and Radiation Survey, dated February 3, 1976.

The licensee has experienced similar leaks in charging pump piping and has initiated PDCR 75-21 to prevent recurrence of weld fatigue failures. The licensee's action concerning this design change are discussed in Detail 9.a of this report.

The records associated with Job Order #76-28 did not indicate the performance of quality assurance inspection of the work or review of the completed job order. The licensee's quality assurance representative was not present during the inspection, and the licensee was unable to demonstrate the completion of these quality assurance requirements.

This is an unresolved item pending review of the quality assurance aspects of Job Order #76-28.

c. Reportable Occurrence 76-02

Reference: Licensee letter to NRC Region I dated March 10, 1976.

This occurrence concerned leakage from a cracked weld in a piping reducer on the discharge header of no. 3 charging pump. There was no detectable increase in air activity in the area of the leakage. The cause and corrective action was the same as discussed in Detail 5.b with the exception that the weld was radiographed following repair. The licensee's actions concerning this occurrence were documented in the following records:

- (1) Maintenance Request #76-140.
- (2) Job Order #76-31.
- (3) OP-5100, Rev. 3, completed February 10, 1976.
- (4) PORC Minutes, Meetings 76-11, 76-12 and 76-19.
- (5) Breathing Zone Air Sample Results for P.A.B. #3 Charging Pump, dated February 10, 1976.
- (6) YAEC Surface Contamination and Radiation Survey, dated February 10, 1976.

The licensee's actions to prevent recurrence of weld fatigue failures are discussed in Detail 9.a of this report.

The records associated with Job Order #76-31 did not indicate the performance of quality assurance inspection of the work or review of the completed job order.

The licensee's quality assurance representative was not present during the inspection, and the licensee was unable to demonstrate the completion of these quality assurance requirements.

This is an Unresolved Item pending review of the quality assurance aspects of Job Order #76-31.

6. Nonroutine Event Review

- a. The licensee's understanding of the following assigned responsibilities associated with nonroutine events was reviewed.
 - (1) Prompt review and evaluation of off-normal operating events to assure identification of safety related events.
 - (2) Prompt review of planned and unplanned maintenance and testing activities to assure identification of noncompliance with the limiting conditions for operation requirements of the Technical Specifications.

- (3) Reporting safety-related operating events internally and to the NRC.
 - (4) Assuring completion of corrective actions relating to safety-related operating events.
- b. The review included discussions with licensee personnel and review of the quality of nonroutine event reports.
 - c. The inspector identified no discrepancies in this area.
7. Safety Limits, Limiting Safety System Settings and Limiting Conditions for Operation
- a. Startup, power or shutdown operations of the following systems were reviewed to verify reactor operations were in conformance with Technical Specification Safety Limits, Limiting Safety System Settings, and Limiting Conditions for Operation:
 - (1) Reactor Coolant System.
 - (2) Power Conversion and Auxiliary Systems.
 - (3) Containment Systems.
 - (4) Emergency Core Cooling Systems.
 - (5) Other Engineered Safety Features.
 - (6) Electrical Systems.
 - b. The review included discussions with licensee personnel, review of Technical Specifications, direct observation of the following process instrumentation and conditions, and review of the following procedures and records:
 - (1) Loop Flow Indications and Annunciator Alarms.
 - (2) OP-6101, Rev. 2, "Nuclear Instrumentation and Reactor Protection System Precritical Checkoff," completed January 31, 1976.

- (3) OP-4606, Rev. 2, completed January 20, 1976, February 17, 1976 and March 18, 1976.
- (4) OP-647, Rev. 1, completed July 21, 1975, August 20, 1975 and September 18, 1975.
- (5) Ion Exchanger Isolation Valve Lineup.
- (6) Ready and Standby Charging Pump Valve Lineup.
- (7) OP-2000.24, Rev. 0, "Preparation and Testing of Hot Leg Insertion System," completed December 3, 1975.
- (8) Shutdown Cooling Systems Valve Lineup.
- (9) Alignment of Charging System for Safety Injection.
- (10) OP-4212, Rev. 3, "Monthly V.C. Recirculation System Valve Exercise," completed March 29, 1975.
- (11) OP-4610, Rev. 1, completed November 11, 1975.
- (12) Offsite Electrical Supply availability.
- (13) Charging System motor operated valves breaker.
- (14) OP-2100, Rev. 5, "Plant Startup from Cold Shutdown," completed January 31, 1976.
- (15) Switching Order 9404U, dated January 31, 1976.
- (16) Accumulator Level and Pressure Indications.
- (17) Rowe Station Logs 1 and 2, January 1, 1976 - March 31, 1976.
- (18) OP-2108, Rev. 2, "Routine Power Operation," dated January 22, 1976.
- (19) OP-2154, Rev. 0, "Operation of Purification System" dated May 2, 1975.
- (20) OP-2163, Rev. 0, "Startup of Purification System," completed December 2, 1975.

c. The inspector determined that the items reviewed and observed were acceptable, except as described below.

- (1) The licensee's most recent startup of the purification system was completed on December 2, 1975, in accordance with OP-2163. The inspector noted that the PORC approved procedure had been changed by the operator performing the procedure to reflect the existing plant conditions. The changes did not change the intent of the procedure or result in an improper valve lineup. However, the changes were not made with the concurrence of two individuals holding senior reactor licenses, and the revised procedure was not subsequently reviewed by PORC and approved by the Plant Superintendent, as required by AP-0001, "Plant Procedures," and Technical Specification D.1.

The failure to maintain proper administrative control of the revised procedure is an Item of Noncompliance, Infraction Level.

8. Control of High Radiation Area Access

References: (1) Region I Inspection Report 50-29/76-06, Detail 6.b
(2) Licensee letter (WYR 76-45) to Region I dated April 12, 1976.

The inspector reviewed the licensee action to review and correct the Item of Noncompliance identified in Reference (1). The inspector verified that the licensee has reviewed the Item of Noncompliance and has taken action to prevent recurrence as described in Reference (2), which consisted of establishing administrative controls for securing all high radiation areas.

This item is resolved.

9. Licensee Followup Actions on Previously Identified Reportable Occurrences

a. Reportable Occurrences 75-07, 76-01 and 76-02

References: (1) Licensee letters to NRC Region I dated August 8 and 15, 1975.

- (2) Region I Inspection Reports 50-29/75-10, Detail 6.b., and 50-29/76-02, Detail 17.
- (3) Licensee letter to NRC Region I dated March 4, 1976.
- (4) Licensee letter to NRC Region I dated March 10, 1976.

The occurrence described in References (1) and (2) concerned a vibration induced crack in a nozzle to flange weld on the charging header. The occurrences described in References (3) and (4) and in Details 5.b and 5.c of this report concern similar weld cracks in charging system piping. The licensee initiated Plant Design Change Request (PDCR) 75-21, "Additions of a Pulsation Dampener on the No. 3 Charging Pump Discharge Header," on September 23, 1975. The materials required to perform the design change have been received with the exception of the pulsation dampener, which is expected to arrive in May, 1976. Based on the timely delivery of the dampener, the licensee expects prefabrication efforts will be completed in July, 1976. The installation of the modified piping section will be completed during the next subsequent maintenance shutdown. This remains an Unresolved Item pending the licensee's completion of the design change.

b. Reportable Occurrence 75-12

- References:
- (1) Licensee letter to NRC Region I dated December 12, 1975.
 - (2) Region I Inspection Report 50-29/76-02, Detail 7.e.
 - (3) Licensee letter to NRC Region I dated March 19, 1976.

The occurrence described in References (1) and (2) concerned the high pressurizer level scram bistable setpoint being greater than the Technical Specification limit. The licensee submitted a revised Licensee Event Report, Reference (3),

which corrected the cause description of the original LER. The licensee stated that a revision to procedure OP-4626 had been drafted and was scheduled for review by the Plant Operations Review Committee. This remains an Unresolved Item pending completion of the licensee's actions to revise OP-4626.

c. Reportable Occurrence 75-17

- Reference: (1) Licensee letter to NRC Region I dated January 2, 1976.
- (2) Region I Inspection Report 50-29/76-02, Detail 7.i.

The occurrence described in References (1) and (2) concerned the setpoint of the Loop No. 1 isolated loop ΔT interlock being set greater than the Technical Specification limit. The licensee stated that the procedure used to adjust this setpoint, OP-6200, would be revised by July 1976 to prevent recurrence of the setpoint drift. The procedure will not be required until the next refueling shutdown which is scheduled for early 1977. This remains an Unresolved Item pending revision of OP-6200.

10. Procedure Changes

- References: (1) Region I Inspection Reports 50-29/75-17, Detail 3.a.(1)(c), and 50-29/76-02, Details 12.b and 12.c.

- a. The licensee has approved revisions to Annunciator Alarm Procedures, which were affected by the recent ECCS modifications. These procedures include OP-3614, OP-3615, OP-3619, OP-3629, OP-3645, OP-3646, OP-3647, OP-3649 and OP-3650. The inspector had no further questions concerning this item.
- b. Procedures associated with the maintenance of motor operated valves have been revised or originated to include instructions for reinstalling MOV cables lifted to comply with the Technical Specifications. These procedures which include OP-5101, OP-5764, OP-5765 and OP-5766, have been approved by the Plant Operations Review Committee, and are awaiting approval by the Manager of Operations. This remains an Unresolved Item pending final approval of the procedures.

11. Switching Log

References: (1) Region I Inspection Reports 50-29/75-10,
Detail 3.c.(1) and 50-29/76-02, Detail 4.c.(1)

The licensee has revised AP-0017, which provides an additional log sheet, APF-0017.4, "Tagging Status Log." This log sheet is used to maintain a record of outstanding equipment tags in the plant. The inspector had no further questions concerning this item.

12. Fire Protection Training

References: (1) Region I Inspection Report 50-29/76-01
Detail 9.

The licensee's implementation of AP-0503, "Fire Protection Training" was reviewed. The licensee conducted annual fire protection general plant training on February 18 and 19, 1976. This training included lectures which covered the location and operation of fire fighting apparatus, communications equipment and breathing equipment. The annual plant fire drill was conducted on March 26, 1976. Deficient areas noted during the drill were scheduled for increased emphasis during future training sessions. The inspector had no further questions concerning this item.

13. Identification of Safety Related Instrumentation and Controls

References: (1) Region I Inspection Report 50-29/74-16,
Detail 13.b.(1) and 50-29/76-02, Detail 28.

The licensee issued a Safety Classification of Systems Manual, Rev. 0, on February 10, 1976. This manual identifies safety-related instruments and controls on the plant, and in addition identifies safety-related mechanical and electrical components. The inspector had no further questions concerning this item. The Safety Classification of Systems Manual will be further reviewed during the next quality assurance inspection.

14. Reported Radiochemistry Data

References: (1) Region I Inspection Report 50-29/76-02,
Detail 5.c.(1).

(2) Licensee letter to NRC, Office of Nuclear
Reactor Regulation, dated March 2, 1976.

The licensee reviewed the radiochemistry data reported in Section V of the January - June 1975 Semi-Annual Operating Report and submitted Reference (2) to correct that report. The inspector noted that the maximum Iodine-131 concentration in the reactor coolant for the month of April 1975 still had not been correctly reported. The activity was listed as 8.9 E-04 vice 9.7 E-04, which was measured on April 29, 1975. The licensee acknowledged this discrepancy. This was the only remaining error in reporting radiochemistry data which was noted by the inspector. Additional licensee review and correction of reported information is not required. The inspector had no further questions concerning this item.

FROM YANKEE ATOMIC ELECTRIC CO NY		DATE OF DOCUMENT 6/7/76	DATE RECEIVED 6/29/76	NO 60729
		LTR X	MEMO	REPORT
TO E (VOLGERMAN)		ORIG X	CC	OTHER
		ACTION NECESSARY <input type="checkbox"/>	CONCURRENCE <input type="checkbox"/>	DATE ANSWERED
		NO ACTION NECESSARY <input type="checkbox"/>	COMMENT <input type="checkbox"/>	BY:
CLASSIF U	POST OFFICE REG. NO.	FILE CODE		
DESCRIPTION (Must Be Unclassified) 50-29/76-11 LETTER IN RESPONSE TO REPORT CONCERNING TWO INFRACTIONS REPORTED BY YOUR INSPECTOR.		REFER TO DREHER	DATE	RECEIVED BY R.A. HARTFIELD
ENCLOSURES COPIES FOR REACTOR FACILITIES BRANCH PDR, LOCAL PDR, NSIC, AND DTIC SENT TO REGIONAL COORDINATOR FOR DISTRIBUTION.				
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U. S. NUCLEAR REGULATORY COMMISSION

MAIL CONTROL FORM

FORM NRC 3265
(1-75)

POOR ORIGINAL