DOCKET NO. 50-29

HAZARDS ANALYSIS BY THE RESEARCH AND POWER REACTOR SAFETY BRANCH

DIVISION OF LICENSING AND REGULATION

IN THE MATTER OF

YANKEE ATOMIC ELECTRIC COMPANY

PROPOSED CHANGE NO. 5

Yankee Atomic Electric Company has submitted Proposed Change No. 5, dated May 1, 1961, which requests a modification in the technical specifications of License No. DPR-3. This modification was requested pursuant to the provisions of paragraph 3.A. of the license and provides for changing the first sentence of paragraph E.2.f.(3) of Appendix A to read as follows:

"As determined at the point of discharge from the primary vent stack and averaged over a period not exceeding one year, the concentrations of radioactive gaseous wastes discharged shall not be in excess of 1,000 times the limits specified in Appendix B, Table II, 10 CFR 20."

The dilution factor for radioactive gaseous wastes is unchanged from that previously provided for in the technical specifications, except in regard to the noble fission gases. The limit stated for noble fission gases in the technical specifications was based on an earlier edition of Title 10 CFR Part 20 which has since been revised. The proposed change provides limitations in accordance with the latest revision.

Based on a review of meteorological data which has been presented by Yankee, including the results of smoke tests performed at the site, we have concluded that an assumed dilution factor of 1000 from the outlet of the 149 foot primary vent stack to the nearest unrestricted area is conservative for radioactive noble fission gases. Although the previous limitations on the discharge of these gases was more restrictive, we believe that the proposed new limits will result in concentrations of radioactive noble gases at the site boundary well within those permitted by Commission regulations, and this modification does not present significant hazards considerations not described or implicit in the license application. We have further concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this modification in the technical specifications.

Edson G. Case, Chief Research and Power Reactor Safety Branch

Date MAY 2 5 1981